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ABSTRACT

The nine Reactor Statics Modules are designed to introduce students to the use of numerical methods and digital computers for calculation of neutron flux distributions in space and energy which are needed to calculate criticality, power distribution, and fuel burnup for both slow neutron and fast neutron fission reactors. The last module, RS-9, includes a separate program, WANDIC (one-dimensional diffusion code with three energy-group representation), for the calculations of neutron spectra and group constants for use in fast breeder reactor calculations. It can be used with the criticality programs in RS-2 and RS-8. The reactor is divided into several concentric annular regions and it is assumed that those regions can be represented fairly well by their average properties. The previous modules show how these properties are found. The set of nine modules is intended to supplement textbooks and other lecture material generally available to students in their course work. It is assumed that students are familiar with elementary nuclear structure, neutron-nuclei interactions, and introductory material on fission chain reactors. (Author/SK)

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REACTOR STATICS MODULE, RS-9
MULTIGROUP DIFFUSION PROGRAM
USING AN EXPONENTIAL ACCELERATION TECHNIQUE

by

Victor C. Macek

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Reactor Statics Module, RS-9

Multigroup Diffusion Program Using an Exponential Acceleration Technique

9.1 Introduction

A reactor physicist or a nuclear engineer designing a power reactor needs to know the number of neutrons present within the system. This information is necessary for the calculation of the power distribution in a reactor from which further vital information on other related physical quantities needed to assure a successful operation of the reactor can be derived.

Previous modules discussed a broad area of problems which are encountered in reactor physics design. The basic input for calculations consists of engineering specifications and nuclear data. The former provide quantities like power density, fuel composition and a geometric description of the reactor. This information varies from one design to another. On the other hand, the nuclear data contains detailed cross-section information.

This input starts a rather long string of calculations before one can obtain results in a form which is suitable for engineering application.

The detailed treatment of space-energy physical phenomena in the entire reactor represents a very difficult task. Therefore, the problem is usually divided into several different stages and the flow of calculations passes from the detailed analysis of the neutron balance in an elementary fuel pin cell to the reactor as a whole consisting of assemblies which are represented by their average properties. In this procedure, the spatial and energy effects are reduced in detail; the former through the "homogenization", the latter through the group condensation. Of course, care must be taken in these simplifications. For example, neutron balance must be conserved.

The code WANDIC (one-dimensional diffusion code with a three energy-group representation) which is described in this module can be used to perform the global reactor calculations. The reactor is divided into several concentric annular regions and it is assumed that we can represent those regions fairly well by their average properties. The previous modules show that these properties are found.

Features of WANDIC

9.2 General Description

The program solves the three group diffusion equations

$$D_g \nabla^2 \phi^g - \Sigma_T^g \phi^g + S^g = 0 \quad \text{for } g = 1, 2, 3 \quad (9.2.1)$$

where the source term consisting of fission neutrons appears in the highest energy group only. The source terms in the lower groups are due to scattered neutrons from the adjacent upper group. The source S^g is given as

$$S^g = \frac{\delta_{1g}}{K} \sum_{g=1}^3 (\nu \Sigma_f)^g \phi^g + \sum_s^{g-1} S^{g-1} \phi^{g-1} \quad (9.2.2)$$

where δ_{1g} is Kronecker's delta; $\delta_{1g} = 1$ for $g = 1$, zero otherwise. The group diffusion equations are solved for cylindrical geometry with θ and β symmetries by using a finite difference technique which is described in module RS-2. Fluxes in each group and each mesh-point are calculated by solving a system of linear equations.

$$\begin{aligned} b_1^g \phi_1^g - a_1^g \phi_2^g &= S_1^g \\ \hline -c_n^g \phi_{n-1}^g + b_n^g \phi_n^g - a_n^g \phi_{n+1}^g &= S_n^g \\ \hline -c_M^g \phi_{M-1}^g + b_M^g \phi_M^g &= S_M^g \end{aligned} \quad (9.2.3)$$

where indices 1, M denote mesh-points at the center and outer boundary of the reactor respectively. S_n is the source at the point n.

In matrix form the system (9.2.3) can be written as

$$A \underline{\phi}^g = \underline{S}^g \quad (9.2.4)$$

where ϕ^g is a vector representing fluxes ϕ_n^g at all mesh-points. The source term is a vector whose components are

$$S_n^g = h_n (\sum_{sn}^{g-1} g_{\phi_n^g} + \frac{\delta_{1g}}{K} \sum_{j=1}^3 (v \sum_f)_{n \phi_n^j})$$

where h_n is volume of the mesh-interval. In matrix notation S_n^g can be written as

$$S^g = E \underline{\phi}^g + \frac{\delta_{1g}}{K} F \underline{\phi}, \quad (9.2.5)$$

where

δ_{1g} is the Kronecker delta and

$$\underline{\phi} = \begin{bmatrix} \phi^1 \\ \phi^2 \\ \phi^3 \end{bmatrix} \quad E^g = (E^{1g} \ E^{2g} \ 0) \\ F = (F^1 \ F^2 \ F^3).$$

The vector ϕ^j is formed from the elements ϕ_n^j , the diagonal matrix E^{jg} is formed from the elements $\sum_{sn}^{g-1} g$, and the diagonal matrix F^j is formed from the elements $(v \sum_f)_{n \phi_n^j}$.

Finally, the assembly of group equations may also be regarded as a matrix problem, which may be written as

$$\underline{\underline{A}} \underline{\phi} = \underline{S}, \quad (9.2.6)$$

where

$$\underline{\underline{A}} = \begin{bmatrix} A^1 & 0 & 0 \\ 0 & A^2 & 0 \\ 0 & 0 & A^3 \end{bmatrix}, \quad \underline{S} = \underline{E} \underline{\phi} + \frac{\delta_1 g}{k} \underline{F} \underline{\phi}$$

$$\underline{F} = \begin{bmatrix} F^1 \\ F^2 \\ F^3 \end{bmatrix}^T, \quad \underline{E} = \begin{bmatrix} E^1 \\ E^2 \\ E^3 \end{bmatrix}^T$$

9.3 The Inner Iteration

The inner iteration allows calculation of spatial dependence of fluxes with respect to the boundary conditions. For simple one-dimensional geometries the inner iteration can be avoided and fluxes at each mesh-point calculated by a direct procedure such as Thomas algorithm which is described in Module RS-2, Chapter 2. This algorithm is used in this code.

9.4 The Outer Iteration

The overall matrix problem to be solved is expressed in Eq. (9.2.6) as

$$\underline{\underline{A}} \underline{\phi} = \underline{S} \quad (9.4.1)$$

in which

$$\underline{S} = \underline{E} \underline{\phi} + \frac{1}{k} \underline{F} \underline{\phi} \quad (9.4.2)$$

The normal procedure is to begin the solution of the group equations in the highest energy group by using a flux guess for the calculation of the fission source. In the highest group there are no slowing-down scattering terms. Proceeding to the next group, the slowing-down part of the source can be calculated from the new fluxes in group 1; since fission source in this model appears only in the highest group, the fission part of the source in group 2 is zero. The calculation can proceed in this fashion to thermal energy. At the end of the outer iteration we have a completely new flux vector, so that the fission source $F\phi$ may be recalculated and the whole process may be repeated.

The equation which must be solved is

$$(A - E)\phi = \frac{1}{k} F\phi. \quad (9.4.3)$$

The iteration process proceeds by defining a vector

$$\psi = \frac{1}{k} F\phi = \frac{1}{k} S \quad (9.4.4)$$

where

$$S = F\phi. \quad (9.4.5)$$

(Note that S denotes the fission source only.)

The iteration is defined by the relation

$$(A - E)\phi^{(p)} = \psi^{(p-1)}. \quad (9.4.6)$$

From Eq. (9.4.5) we can write

$$S^{(p)} = F\phi^{(p)}. \quad (9.4.7)$$

By combining Eqs. (9.4.5) and (1.4.7), the p-th iterate of the source reactor is given by

$$\frac{1}{k(p)} S^{(p)} = \psi^{(p-1)}. \quad (9.4.7)$$

To estimate the eigenvalue one utilizes the fact that when a convergence is being approached, the fission source for one iteration should equal the fission source for the next. In this code the estimate of the eigenvalue is determined in the following fashion. The iteration starts with $k^{(0)}$ equal to 1 and $S^{(0)}$ such that

$$\int_{(\text{vol of core})} S^{(0)} dv = 1 \quad (9.4.8)$$

Then from (1.4.7) and (1.4.4),

$$k^{(1)} = \int_{(\text{vol of core})} S^{(1)} dv, \quad (9.4.9)$$

and similarly

$$k^{(p)} = \int S^{(p)} dv. \quad (9.4.9)$$

The iteration continues until convergence of the eigenvalue $k^{(p)}$ is obtained, i.e.,

$$\left| \frac{k^{(p)} - k^{(p-1)}}{k^{(p)}} \right| < \varepsilon. \quad (9.4.10)$$

The single power iteration described above tends to converge rather slowly and acceleration techniques are used to improve the convergence rate. If we denote the unaccelerated fission source for iteration p by $s^{*(p)}$, then we calculate an accelerated fission source from

$$s^{(p)} = s^{(p-1)} + \omega(s^{*(p)} - s^{(p-1)}). \quad (9.4.11)$$

This form of acceleration is referred to as a first order acceleration. The Chebyshev polynomial method is described Reactor Statics Module, RS-8. The optimum value of ω requires a knowledge of the maximum eigenvalue of the matrix $(A - E)F^{-1}$. Generally this eigenvalue is not known. One of the methods to solve this problem is described in the following section.

9.5 The Exponential Over-Relaxation Technique

The program uses a special technique for the calculation of the source introduced for the subsequent iteration step.

Fission sources are computed from the calculated fluxes at each mesh point as follows

$$S_n^* = \sum_j (\nu \Sigma_f)_n^n \phi_j \quad j = 1, 2, 3 \\ n = 1, 2, \dots . \quad (9.5.1)$$

For the first iteration the initial source-guess will be, e.g.,

$$S^{(0)} = \frac{1}{\pi R^2} , \quad (9.5.2)$$

where R is the radius of the core. The source for the p -th iteration will be given by formula

$$S_n^{(p)} = S_n^{(p-1)} + \omega(S_n^* - S_n^{(p-1)}) , \quad (9.5.3)$$

where ω is the relaxation factor whose optimal value lies between 1 and 2.

The equation (9.5.3) can be rewritten as

$$S_n^{(p)} = S_n^{(p-1)} [1 + \omega \left(\frac{S_n^*}{S_n^{(p-1)}} - 1 \right)] \\ (9.5.4)$$

$$S_n^p \approx S_n^{(p-1)} \exp \left[\omega \left(\frac{S_n^*}{S_n^{(p-1)}} - 1 \right) \right]$$

for $\left(\omega \frac{S_n^*}{S_n^{(p-1)}} - 1 \right) \ll 1$.

The optimal value of the relaxation factor ω is automatically computed during the calculation.

Letting,

$$Q' = \frac{S^*}{\frac{S_n}{n}^{(p-1)}} - 1, \quad (9.5.5)$$

we can have two cases:

1) $Q' > 0$.

The exponential function can be expressed as

$$\begin{aligned} \exp[\omega(\frac{S^*}{\frac{S_n}{n}^{(p-1)}} - 1)] &\approx 1 + \omega(\frac{S^*}{\frac{S_n}{n}^{(p-1)}} - 1) \\ &\approx 2 - (1 - \omega(\frac{S^*}{\frac{S_n}{n}^{(p-1)}} - 1)) \end{aligned} \quad (9.5.6)$$

$$\approx 2 - e^{-Q},$$

where

$$Q = \omega * \left| \frac{\frac{S^*}{S_n}}{\frac{S_n}{n}^{(p-1)}} - 1 \right|. \quad (9.5.7)$$

2) $Q' < 0$.

In this case we have

$$\exp[\omega(\frac{S^*}{\frac{S_n}{n}^{(p-1)}} - 1)] = e^{-Q}. \quad (9.5.8)$$

The calculation of the exponential function in the core is speeded up by the following approximations:

$$\text{For } Q > 0.1 \quad e^{-Q} \rightarrow \exp[-Q]$$

$$0.001 < Q < 0.1 \quad e^{-Q} \rightarrow \frac{1 - Q/2}{1 + Q/2}$$

$$Q < 0.001 \quad e^{-Q} \rightarrow 1 - Q .$$

In order to find the optimum value of ω it is necessary to scan the maximum relative error at each iteration given by the expression

$$\varepsilon^{(p)}(S) = \max_i \left(\frac{s_i^{(p)} - s_i^{(p-1)}}{s_i^{(p-1)}} \right) , \quad (9.5.9)$$

where i runs through all mesh-points. The computation is normally started with ω equal to 1.9. At the end of 20 iterations the convergence ratio is tested. If the convergence is fast enough, let us say, if

$$\frac{\varepsilon^{(10)}(S)}{\varepsilon^{(20)}(S)} > 1.5 , \quad (9.5.10)$$

the iteration with unchanged ω is continued for another 20 iterations. If the convergence is slow with respect to the above criterion, i.e., the inequality is reversed and if the relative point-wise error oscillates which can be described by the inequality

$$\left| \frac{\varepsilon^{(20)}}{\varepsilon^{(19)}} [1 + |\varepsilon^{(19)}(S)|] \right| > 1.0 , \quad (9.5.11)$$

ω is reduced by 0.1 and with this new value of ω the iteration process is continued in the same manner as described above.

9.6 Accuracy Criteria

In this code two criteria for the outer iteration are adopted. The convergence of the multiplication factor k will be reached when the following holds.

$$\left| \frac{k^{(p)} - k^{(p-1)}}{k^{(p)}} \right| < \frac{\epsilon_1}{5.0} , \quad (9.6.1)$$

where $k^{(p-1)}$ is the eigenvalue obtained in the previous iteration. Once criticality is obtained, the program proceeds to obtain the pointwise convergence of neutron group fluxes which will be reached when the following condition is fulfilled.

$$\left| \frac{\phi_n^{(p)} - \phi_n^{(p-1)}}{\phi_n^{(p-1)}} \right|_{\max} < \epsilon_2 . \quad (9.6.2)$$

With the poison search another convergence criterion must be included which is discussed further.

9.7 Control Search

The program does a control search on a poison or any fictitious material which adjusts the reactor to the critical condition. Let the multiplier of the poison concentration be denoted by X . In general, the parameter X can be applied to any other type of control. By changing X the program finds k , then changes X , finds k again, and so on. In the early stages of the process the k calculations stop when both criteria (9.6.1) and (9.6.2) are satisfied. Then the criterion to guarantee criticality is applied, i.e.,

$$|k(\text{converged}) - k_c| < \varepsilon_1, \quad (9.7.1)$$

if k_c is chosen to be unity, the reactor is critical. Clearly k_c also can be chosen to take into account the presence of other reactivity controls.

The initial guess for the multiplication factor proceeds according to the following formulas:

- a) If the first guess for the multiplication factor X is zero, the second guess is

$$X_2 = \Delta X * \{\text{of sign of } (1.0 - k_1)\} . \quad (9.7.2)$$

- b) If the first guess is non-zero the second guess is given by

$$X_2 = 1.0 + \Delta X * [\text{sign of } (1.0 - k_1)] * X, \quad (9.7.2)$$

where k is the converged eigenvalue and ΔX is an increment in the

parameter X. This formula tries to ensure that X moves in the right direction: it assumes that if X is negative, an increase of X decreases the reactivity, and the converse of this if X is positive (this would be applied, e.g., for the search of fissile isotopic composition). The programs finishes when three conditions (9.6.1), (9.6.2), and (9.7.1) are satisfied.

With the quantities X_1 and X_2 the search is begun and X is varied in order to make k equal to unity. It is easy enough then to make a simple, search procedure such as ordinary "regula falsi," but these processes appear to be very inefficient in this context. The difficulty is that given X, we can find k but the more accurately we require it, the greater the number of iterations is required. One way to go is to improve "regula falsi" process so that the search is accelerated. In ordinary "regula falsi" at each stop, it is one of the limits (a, b) which is used and the last approximation of the root. Instead of that we can use the last two approximations which are closer to the root which is searched for than the limits of the interval (a, b). The formula which is used has the form given by Equation (9.7.5) and is illustrated in Figure 1.

$$a_{n+1} = a_n - f(a_n) * \frac{a_n - a_{n-1}}{f(a_n) - f(a_{n-1})} . \quad (9.7.4)$$

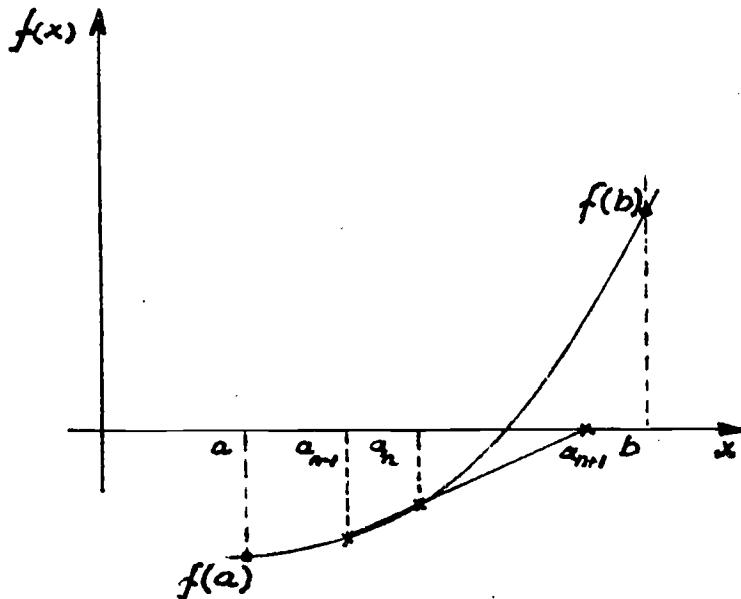


Fig. 1.

The a_1 is found from the formula

$$a_1 = b - f(b) \frac{b - a}{f(b) - f(a)} . \quad (9.7.5)$$

Depending on the sign of $f(a)$, $f(b)$, and $f(a_1)$, a_2 is computed from the formula

(9.7.6) if $f(a) < 0$, $f(b) > 0$ and $f(a_1) < 0$; if $f(a_1) > 0$ from the formula (9.7.7),

$$a_2 = a_1 - f(a_1) \frac{b - a_1}{f(b) - f(a_1)} \quad (9.7.6)$$

and

$$a_2 = a_1 - f(a_1) \frac{a_1 - a}{f(a_1) - f(a)} . \quad (9.7.7)$$

If perchance the point a_3 will appear outside the interval (a, b) then in the following step it will be necessary to consider the closer limit instead of the point a_3 . It can be shown that convergence of this method is faster than the ordinary "regular falsi".

9.8 Power Normalization of Neutron Fluxes

The source-normalized power at the point \vec{r} is given by the expression

$$p(\vec{r}) = \sum_1^3 \kappa_f^j \Sigma_f^j(\vec{r}) \Phi_j^j(\vec{r}) \quad j = 1, 2, 3 \quad (9.8.1)$$

where κ_j stands for energy produced per fission reaction, $\Phi_j^j(r)$ is the source-normalized flux. In order to get fluxes normalized to the total power output of the reactor we have to multiply the source normalized fluxes by the normalization factor γ given by the formula

$$\gamma = \frac{P|H}{\int_0^R p(\vec{r}) dr} , \quad (9.8.2)$$

where $dr = 2\pi r dr$ and H is the height of the reactor. The power-normalized flux Φ'_j is then given by

$$\Phi'_j(\vec{r}) = \gamma \Phi_j^j(r) . \quad (9.8.3)$$

9.9 Organization of the Program

The code WANDIC has the following limitations:

- a) Number of energy groups is three
- b) The reactor can be divided into four regions from which the outer one is non-multiplicative. If use of fewer regions is desired, the entry data for some of these regions will be identical.
- c) Memory array space is provided for up to 300 mesh-points.
- d) Number of intervals dividing each region must be even.

The code consists of a number of subprograms which perform different parts of calculation and are controlled by the main program.

9.10 The Role of Different Subprograms

PLOT: The subprogram participating in preparation of the output and plotting the related results on the printer.

SIMP: The subprogram performing the integration of functions over the core volume by SIMQSON'S rule.

ZERO: The subprogram which employs the accelerated "regula falsi" technique.

9.11 Flow-Chart

Besides its role as the manager of subprograms, the main program is in charge of entry of data for calculation of fluxes and preparation of results for the output. In Figure 2 the flow-chart of the sequence of calculation is presented.

FLOW-CHART A

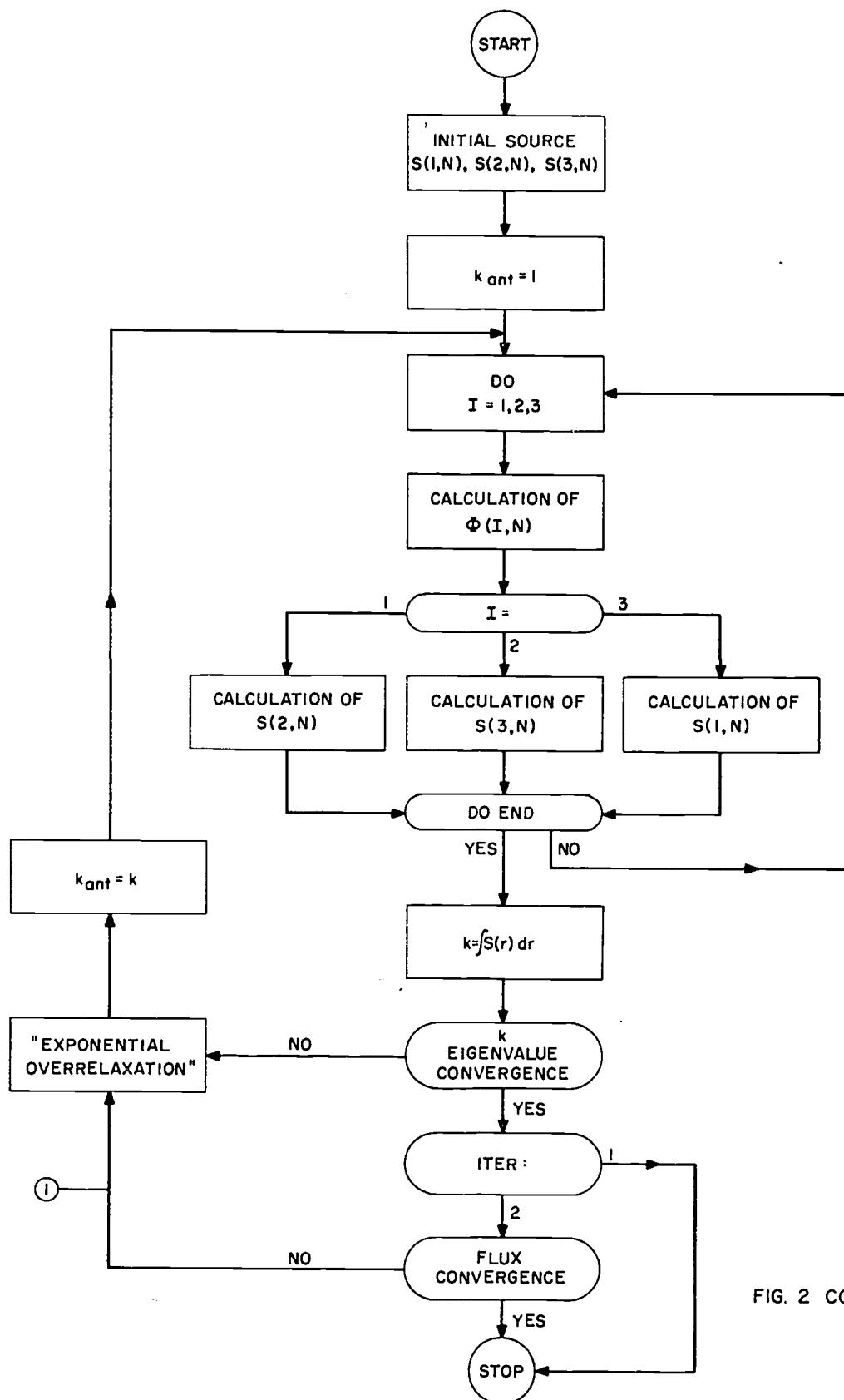
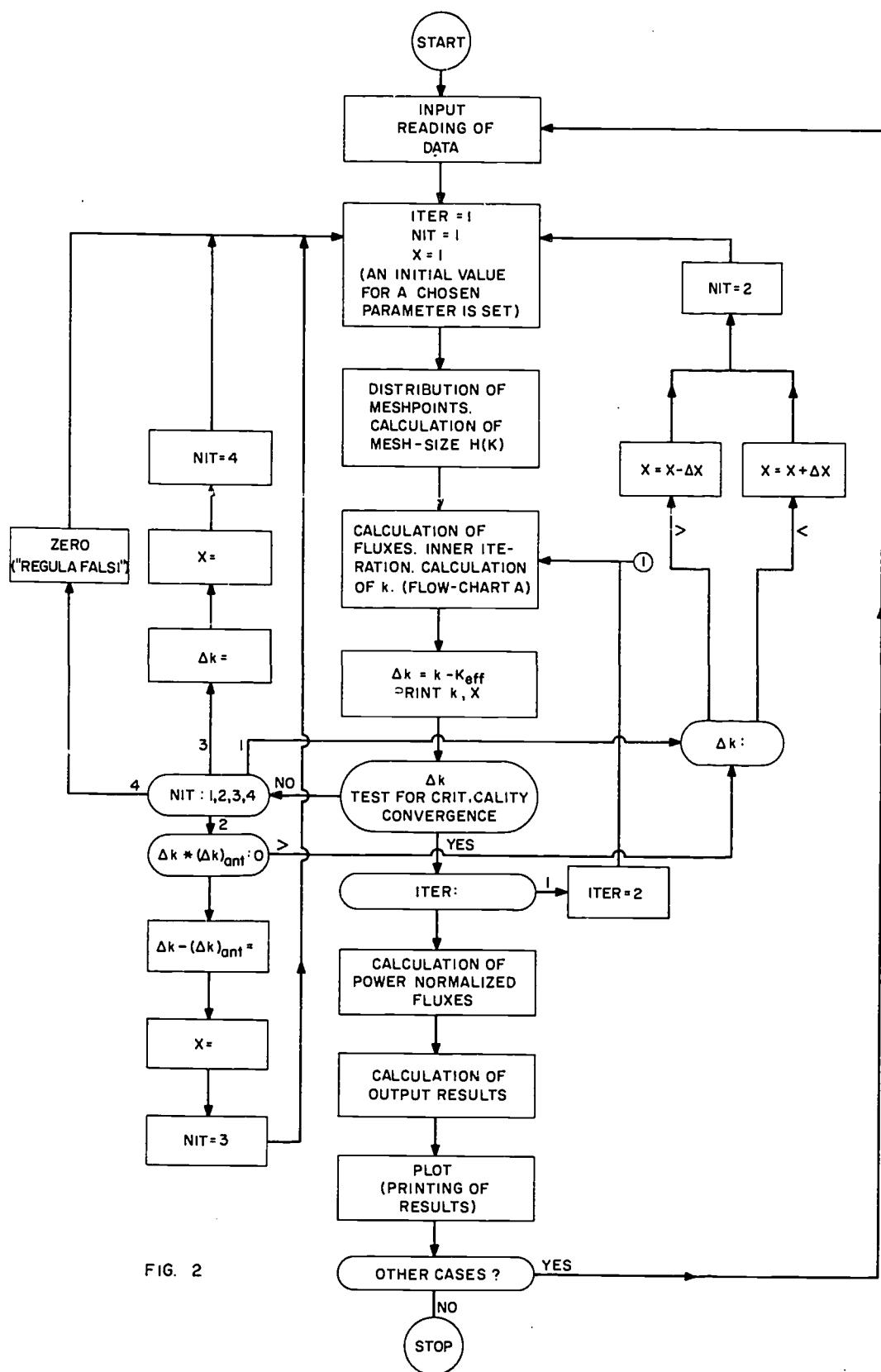


FIG. 2 CON'T.

GENERAL FLOW-CHART OF THE PROGRAM



9.12 Output

The output of the program are:

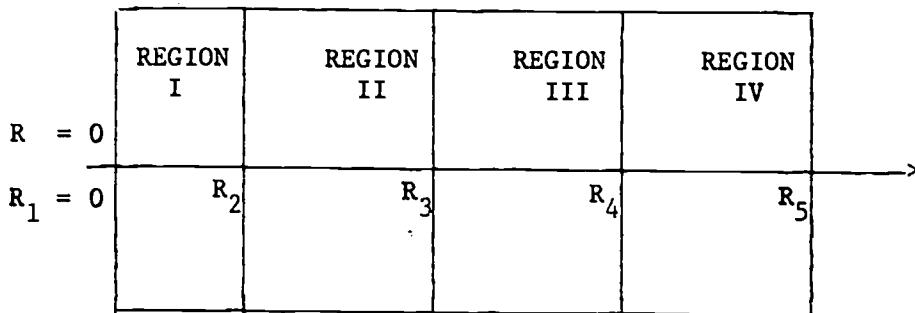
- a) source normalized group fluxes versus radius
- b) relative magnitudes of fluxes at each mesh-point with respect to the flux at the symmetry axis of the core
- c) ratio of group fluxes
- d) eigenvalue of the unpoisoned reactor
- e) poison concentration needed for criticality
- f) power normalized group fluxes
- g) power generated in each fueled region with respective power densities
- h) for user who wants to watch the iteration process, printing of eigenvalue K computed at each iteration is provided.

9.13 Names of Principal Variables Used by the Program

- H(K): mesh-size of the region K
- FLUX(I,N): neutron flux of the group I at the mesh-point N
- SDS(N): slowin_-down source at the mesh-point N
- FS(N): fission source at the mesh-point N
- S(N): source at the mesh-point N
- EPS(P): maximum relative error for the source on the p-th iteration
- FME: $K_c = 1$
- XP: multiplication factor for the control poison cross-section
- FMANT: eigenvalue of the preceding iteration
- FPR(K): power density of the region K.

9.14 Use of the Program for a Simple Problem. Presentation of Entry of Data

The data introduced here are for illustration purposes only. Consider a cylindrical reactor consisting of three different fissionable regions and a reflector (see figure below). The outer radius of the first region is $R_2 = 87.7$ cm, the second $R_3 = 124.00$ cm, $R_4 = 152.00$ cm, $R_5 = 159$ cm.



The data for the problem would start:

1 card: symbols used for reading are

OVREL CONV PREC BH2 SCP POW NINT(1) NINT(2) NINT(3)
NINT(4) ICP ICC

where

OVREL: the relaxation factor (Sec. 1.5). This factor for a new problem is put equal to 1.9. Once an "optimum" factor is found, this may be introduced into a set of similar problems (e.g., with the same mesh-point distribution).

CONV: the pointwise flux convergence accuracy number (ϵ_2). (See Eq. (9.6.2))

PREC: the accuracy (ϵ_1) for the eigenvalue convergence. (See Eq. (9.6.1)).

BH2: axial buckling.

SCP: microscopic cross-section of a poison material for the third group energy level in barns.

POW: thermal output of the reactor in watts.

NINT(K): number of intervals per region K.

ICP: control search parameter. If its value is
1...control poison is considered in region 1
2...region 2
3...region 3
4...all fueled regions
5...regions 1 and 2
6...regions 2 and 3

ICC: is the case control parameter (equal to zero, if only one case
is to be treated; positive, if otherwise)

The three group diffusion equations in one of the multiplicative powers
assumes, in general, the following form

$$D_1 \Delta^2 \phi_1 - (\sum_{a_1} + \sum_{S_1 1 \rightarrow 2}) \phi_1 + \sum_{i=1}^3 \frac{(v \sum_f)_i \phi_i}{k} = 0$$

$$D_2 \Delta^2 \phi_2 - (\sum_{a_2} + \sum_{S_1 2 \rightarrow 3}) \phi_2 + \sum_{S_1 1 \rightarrow 2} \phi_1 = 0$$

$$D_3 \Delta^2 \phi_3 - \sum_{a_3} \phi_3 + \sum_{S_1 2 \rightarrow 3}^1 \phi_2 = 0$$

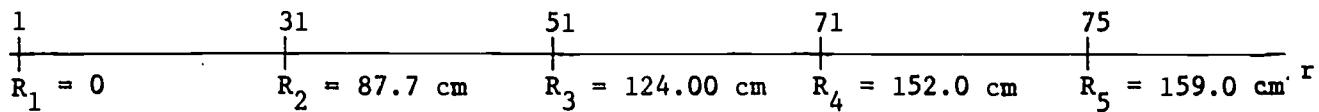
For example, with calculated macroscopic cross-sections

$$1.7 \Delta^2 \phi_1 - (0.004 + 0.06) \phi_1 + \frac{1}{k} (0.43 \phi_1 + 0.015 \phi_2 + .16 \phi_3) = 0$$

$$0.85 \Delta^2 \phi_2 - (0.025 + 0.05) \phi_2 + 0.06 \phi_1 = 0$$

$$0.39 \Delta^2 \phi_3 - 0.13 \phi_3 + 0.05 \phi_2 = 0$$

Let us divide the region I into 30 equal intervals, the regions II and III into 20 intervals and region IV into 4 intervals. The mesh-points in the radial direction will be numbered as shown in the figure below.



Let us assume that the axial buckling BH2 = $1.56 * 10^{-3} \text{ cm}^2$; we shall use poison search in all fissionable regions, ICP = 4, the microscopic absorption cross-section of the poison material $\sigma = 2109 \text{ bn}$, and this will be the only case to be treated. The power output of the reactor is 2441 MW.

OVREL	CONV	PREC	BH2	SCP	POW
1.9E - 01	1.E - 02	1.E - 04	156.E - 05	2109.E 00	2441.E + 06
N ₁	N ₂	N ₃	N ₄	ICP	ICC
30	20	20	04	04	0

2, 3, 4th Cards:

The second card carries physical data for the first region--group 1, the third those for group 2, the fourth for group 3. Data are read in the following order (the symbols are self-explanatory; $\frac{\text{watts}}{\text{fission}}$).

$$D_j \quad \Sigma_{aj} \quad \Sigma_{r_1 j \rightarrow j+1} \quad (v\Sigma_f)_j \quad (\Sigma_f)_j$$

2nd:	17.E - 1	4.E - 3	6.E - 2	43.E - 2	93.E - 15
3rd:	85.E - 2	25.E - 3	5.E - 2	1.E - 2	19.E - 14
4th:	39.E - 2	13.E - 2	0.E + 0	16.E - 2	21.E - 13

Other 6 cards carry the similar data for regions II and III.

11, 12, 13th Cards:

These cards carry the data for the outer non-fissionable region in order of groups 1, 2, 3. These data are

$$D_j \quad \Sigma a_j \quad \Sigma S_{j+j+1}$$

Let the reflector have the following data in order of groups 1, 2, 3

11th:	19.E - 1	36.E - 5	26.E - 3
12th:	92.E - 2	57.E - 5	28.E - 3
13th:	31.. - 2	85.E - 4	0.E 0

14th Card:

This card carries outer radii of regions with increasing order

R ₂	R ₃	R ₄	R ₅
877.E - 1	124.E 0	152.E 0	159.E 0

15th Card:

The value carried by this card is the increment of the multiplier adjusting the poison concentration at early stages of control search.

PP

0.05

9.15 Rules for Card Punching

The format of data read is fixed: real variables are read with the format E 10.0, the integer ones with the format I2 with one blank space separating them. On the last card PP, variable is read with the format F5.3.

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```

C      MAIN PROGRAM          A   1
C      PROGRAM HANDIC        A   2
C
C
C      THE PROGRAM SOLVES THE THREE-GROUP DIFFUSION EQUATIONS    A   5
C      FOR A CYLINDRICAL REACTOR CONSISTING OF THREE FUELED REGIONS A   6
C      AND A REFLECTOR       A   7
C
C
C      INPUT-MEANING OF SYMBOLS          A   9
C
C      NINT...NUMBER OF INTERVALS FOR EACH REGION          A  10
C      OVREL...OVERRELAXATION FACTOR           A  11
C      CONV...PRECISION FOR THE POINTWISE FLUX CONVERGENCE     A  12
C      X...MULTIPLIER FOR THE CONTROL SEARCH PARAMETER     A  13
C      D...DIFFUSION COEFFICIENT            A  14
C      SCA...MACROSCOPIC ABSORPTION CROSS-SECTION      A  15
C      XNF...MACROSCOPIC FISSION CROSS-SECTION*NU       A  16
C      XKF...MACROSCOPIC FISSION CROSS-SECTION*KAPPA     A  17
C      SR...SCATTERING CROSS-SECTION INTO THE LOWER ADJACENT GROUP A  18
C      BH2...TRANSVERSE BUCKLING             A  19
C      RR...OUTER RADIUS OF THE REGIONS 1,2,3,4       A  20
C      POW...POWER OUTPUT OF THE REACTOR         A  21
C      SCP...MICROSCOPIC CROSS-SECTION OF THE CONTROL POISON A  22
C      ICP...CONTROL PARAMETER FOR THE POISON SEARCH      A  23
C      1...REGION 1                         A  24
C      2...REGION 2                         A  25
C      3...REGION 3                         A  26
C      4...ALL FUELED REGIONS             A  27
C      5...REGIONS 1&2                     A  28
C      6...REGIONS 2&3                     A  29
C      ICC...PROGRAM CONTROL INDICATING THE NUMBER OF CASES A  30
C          POSITIVE INTEGER...SEVERAL CASES TO BE TREATED A  31
C          0...ONLY ONE CASE                A  32
C
C
C      LIMITATIONS OF THE CODE          A  33
C
C      1ST:NINT(K) MUST BE EVEN NUMBER      A  34
C      2ND: TOTAL NUMBER OF MESH-POINTS BE LESS THAN 301      A  35
C
C
0001    SUBROUTINE PLOT (LI,N,LP)          A  36
0002    REAL MX,MN,PRNT(6,9),DISP(6),DY,X(300),Y(3,300),H(4)      A  37
0003    INTEGER LI,LP,PT,STRG(101),BLK,PER(21),AST,I,N,NINT(4)      A  38
0004    COMMON Y,H,NINT                  A  39
0005    DATA PRNT/4HFAST,4H FLU,4HX - ,4HGROU,4HP 1 ,1H ,4HRF'S0,4HNANC,4HE      A  40
1     FL,4HUX -,4HGROU,4HP 2 ,4HSLOW,4H FLU,4HX - ,4HGRC',4HP 3 ,1H ,4H      A  41
2     2GROU,4HP 1 ,1H ,1H ,1H ,4HGROU,4HP 2 ,1H ,1H ,1H ,4HGROU,4      A  42
3     3HP 3 ,1H ,1H ,1H ,4HGROU,4HP 1 ,4H/ GR,4HOUP ,4H3 ,1H ,4HGRO      A  43
4     4U,4HP 2 ,4H/ GR,4HOUP ,4H3 ,1H ,4HGROU,4HP 1 ,4H/ GR,4HOUP ,4H2      A  44
5     5 ,1H /,8LK,AST/1H ,1H*/,PER/21*'.*/      A  45
0006    DO 1 I=1,101                   A  46
0007    1     STRG(I)=BLK              A  47
C
C      CALCULATION OF RADII CORRESPONDING TO THE MESH-POINTS      A  48
C
0008    X(1)=0.0                      A  49
0009    K=1                          A  50

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0010      DO 2 I=2,N          A 59
0011      X(I)=X(I-1)+H(K)   A 60
0012      IF (I.EQ.NINT(K)) K=K+1 A 61
0013      2      CONTINUE     A 62
0014      WRITE (6,9) (PRNT(I,LP),I=1,6) A 63
0015      MN=Y(LI,1)          A 64
0016      MX=MN              A 65
0017      DO 6 I=2,N          A 66
0018      IF (MX-Y(LI,I)) 3,4,4 A 67
0019      3      MX=Y(LI,I)   A 68
0020      GO TO 6             A 69
0021      4      IF (MN-Y(LI,I)) 6,6,5 A 70
0022      5      MN=Y(LI,I)   A 71
0023      6      CCNTINUE    A 72
0024      :      DY=(MX-MN)/100 A 73
0025      :      DO 7 I=1,6     A 74
0026      7      DISP(I)=MN+(I-1)*20*DY A 75
0027      :      WRITE (6,10) DISP       A 76
0028      :      WRITE (6,11) (PER(I),I=1,21) A 77
0029      :      DO 8 I=1,N     A 78
0030      :      PT=(Y(LI,I)-MN)/DY+1.5 A 79
0031      :      STRG(PT)=AST      A 80
0032      :      WRITE (6,12) X(I),STRG A 81
0033      8      STRG(PT)=BLK    A 82
0034      :      RETURN        A 83
0035      C
0036      9      FORMAT (1H1,45X,7H PLOT OF,1X,6A4,/,57X,6HZ-AXIS) A 84
0037      10     FORMAT (11X,6(E10.3,10X))           A 85
0038      11     FORMAT (16X,21(A1,4X),/,7H R-AXIS)      A 86
0039      12     FORMAT (E11.3,5X,10A1)          A 87
0039      END          A 88

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```
C      INTEGRATION SIMPSON FOR N POINTS. N MUST BE EVEN.          B   1
C      N MUST BE WITHIN 4 AND 100                                     B   2
0001    FUNCTION SIMP (DX,XX,N,J1)                                    B   3
0002    DIMENSION XX(100), DX(4)                                      B   4
0003    SIMP=XX(1)+XX(N)+4.*XX(N-1)                                  B   5
0004    J=(N-3)/2                                                    B   6
0005    DO 1 I=1,J                                                 B   7
0006    1    SIMP=SIMP+4.*XX(2*I)+2.*XX(2*I+1)                      B   8
0007    SIMP=SIMP*DX(J1)/3.                                         B   9
0008    RETURN                                                       B  10
0009    END                                                          B  11
```

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C ACCELERATED 'REGULA FALSI'
C
C
C
0001      SUBROUTINE ZERO (A1,A2,F1,F2,A10,A20,DF1,DF2,RAD)
0002      1 RAD=A2-F2*(A2-A1)/(F2-F1)
0003          VARA=A2-F2*(A2-A1)/(F2-F1)
0004          HH=ABS(A20-A10)
0005          H1=ABS(VARA-A10)
0006          H2=ABS(VARA-A20)
0007          IF (H1-H2) 2,8,3
0008      2 TEMP=H2
0009          L=2
0010          GO TO 4
0011      3 TEMP=H1
0012          L=1
0013      4 IF (TEMP-HH) 8,5,5
0014      5 GO TO (6,7), L
0015      6 A2=A20
0016          F2=DF2
0017          GO TO 1
0018      7 A2=A10
0019          F2=DF1
0020          GO TO 1
0021      8 RETURN
0022      END

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C ENTRY OF DATA AND DIMENSIONS OF ARRAYS D 1
0001   DIMENSION NINT(4), RR(5), H(4), EPS(30), SCA(3,3) D 2
0002   DIMENSION SCA(3,4), SR(3,4), D(3,4), XNF(3,3), XKF(3,3) D 3
0003   DIMENSION SANT(300), DFMAX(3), FS(300), SUM(300), AIDD(3 D 4
     1,300) D 5
0004   DIMENSION ALFA(300), BETA(300), S(300), FLUX(3,300), GX(100), FPR( D 6
     13) D 7
0005   COMMON AIDD,H,NINT D 8
0006   1 READ (5,142) OVREL,CONV,PREC,BH2,SCP,POW,NINT,ICP,ICC D 9
0007   READ (5,143) ((D(I,K),SCA(I,K),SR(I,K),XNF(I,K),XKF(I,K),I=1,3),K= D 10
     11,3) D 11
0008   READ (5,144) (D(I,4),SCA(I,4),SR(I,4),I=1,3) D 12
0009   READ (5,145) (RR(K),K=2,5) D 13
0010   READ (5,146) PP D 14
C D 15
C PRINTING OF DATA D 16
C D 17
0011   RR(1)=0.0 D 18
0012   WRITE (6,147) D 19
0013   WRITE (6,148) D 20
0014   WRITE (6,149) (RR(L),L=1,5) D 21
0015   WRITE (6,150) NINT,OVREL,PREC,CONV D 22
0016   WRITE (6,151) BH2,SCP D 23
0017   WRITE (6,152) ((SCA(I,K),I=1,3),K=1,4) D 24
0018   WRITE (6,153) ((D(I,K),I=1,3),K=1,4) D 25
0019   WRITE (6,154) ((SR(I,K),I=1,3),K=1,4) D 26
0020   WRITE (6,155) ((XNF(I,K),I=1,3),K=1,3) D 27
0021   WRITE (6,156) ((XKF(I,K),I=1,3),K=1,3) D 28
0022   WRITE (6,157) POW D 29
0023   WRITE (6,158) D 30
C D 31
C NUMBERING OF MESH-POINTS D 32
C D 33
0024   PI=3.14159265 D 34
0025   CF=1.6021E-13 D 35
0026   TEMP=1.0 D 36
0027   ITER=1 D 37
0028   NIT=1 D 38
0029   NINT(1)=NINT(1)+1 D 39
0030   N1=NINT(1) D 40
0031   DO 2 K=2,4 D 41
0032   2 NINT(K)=NINT(K)+NINT(K-1) D 42
0033   N2=NINT(2) D 43
0034   N3=NINT(3) D 44
0035   N3M1=N3-1 D 45
0036   N4=NINT(4) D 46
0037   N4M1=N4-1 D 47
0038   XP=1. D 48
0039   JJ=1 D 49
0040   DO 3 K=1,4 D 50
0041   DO 3 I=1,3 D 51
0042   3 SCA(I,K)=SCA(I,K)+SR(I,K)+BH2*D(I,K) D 52
0043   DO 4 K=1,3 D 53
0044   4 SCA(3,K)=SCA(3,K) D 54
0045   DO 5 N=1,N4M1 D 55
0046   FS(N)=0.0 D 56
0047   5 SUM(N)=0.0 D 57
C D 58

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0094      24   K=K+1                               D 117
0095          R=RR(K)                            D 118
0096          TEMP=(H(K-1)+H(K))*0.5            D 119
0097          AN=(1.+H(K)/(2.*R))*D(I,K)/(H(K)*TEMP) D 120
0098          CN=(1.-H(K-1)/(2.*R))*D(I,K-1)/(H(K-1)*TEMP) D 121
0099          BN=AN+CN+(SCA(I,K)*H(K)+SCA(I,K-1)*H(K-1))/(H(K)+H(K-1)) D 122
0100      25   TEMP=BN-CN*ALFA(N)                D 123
0101          ALFA(N+1)=AN/TEMP                D 124
0102      26   BETA(N+1)=(S(N)+CN*BETA(N))/TEMP  D 125
0103          BM=CN*(3.+H(4)/(1.066*D(I,4)))-AN    D 126
0104          CM=4.*CN-BN                      D 127
0105          GO TO (28,27), ITER               D 128
0106      27   TEMP=FLUX(I,N4)                 D 129
0107      28   FLUX(I,N4)=(S(N4M1)+CM*BETA(N4))/(BM-CM*ALFA(N4)) D 130
0108          GO TO (30,29), ITER               D 131
0109      29   DFMAX(I)=ABS((TEMP-FLUX(I,N4))/TEMP) D 132
0110      30   DO 35 N=1,N4M1                  D 133
0111          L=N4-N                      D 134
0112          GO TO (32,31), ITER               D 135
0113      31   TEMP=FLUX(I,L)                 D 136
0114      32   FLUX(I,L)=ALFA(L+1)*FLUX(I,L+1)+BETA(L+1) D 137
0115          GO TO (35,33), ITER               D 138
0116      33   DFMAX(I)=ABS((TEMP-FLUX(I,L))/TEMP) D 139
0117          IF (DFMAX(I)-TEMP) 34,35,35        D 140
0118      34   DFMAX(I)=TEMP                  D 141
0119      35   CONTINUE                     D 142
0120          C                                D 143
0121          C      CALCULATION OF SOURCE FOR THE FOLLOWING GROUP D 144
0122          C                                D 145
0123          K=1                           D 146
0124          DO 58 N=1,N4M1                  D 147
0125          IF (N.GT.NINT(3)) GO TO 40       D 148
0126          IF (N-NINT(K)) 36,40,36        D 149
0127          36   GO TO (38,38,37), I         D 150
0128          37   SDS(N)=0.0                  D 151
0129          38   GO TO 39                   D 152
0130          39   SDS(N)=SR(I,K)*FLUX(I,N)    D 153
0131          40   FS(N)=XNF(I,K)*FLUX(I,N)    D 154
0132          41   GO TO (43,43,42), I         D 155
0133          42   SDS(N)=0.0                  D 156
0134          43   TEMP1=0.0                  D 157
0135          44   GO TO 44                   D 158
0136          45   SDS(N)=SR(I,K)*FLUX(I,N)    D 159
0137          46   TEMP1=SR(I,K-1)*FLUX(I,N)    D 160
0138          47   TEMP2=XNF(I,K-1)*FLUX(I,N)    D 161
0139          48   FS(N)=0.0                  D 162
0140          49   GO TO 49                   D 163
0141          50   SDS(N)=0.0                  D 164
0142          51   TEMP1=0.0                  D 165
0143          52   GO TO 48                   D 166
0144          53   SDS(N)=SR(I,K)*FLUX(I,N)    D 167
0145          54   TEMP1=SR(I,K-1)*FLUX(I,N)    D 168
0146          55   FS(N)=XNF(I,K)*FLUX(I,N)    D 169
0147          56   TEMP2=XNF(I,K-1)*FLUX(I,N)    D 170
0148          57   SDS(N)=0.0                  D 171
0149          58   TEMP1=0.0                  D 172
0150          59   GO TO 49                   D 173
0151          60   SDS(N)=0.0                  D 174

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 0149 49 TEMP=H(K)+H(K-1) D 175
 0150 SDS(N)=(SDS(N)*H(K)+TEMP1*H(K-1))/TEMP D 176
 0151 FS(N)=(FS(N)*H(K)+TEMP2*H(K-1))/TEMP D 177
 0152 GO TO 53 D 178
 0153 50 GO TO (52,52,51), I D 179
 0154 51 SDS(N)=0.0 D 180
 0155 GO TO 54 D 181
 0156 52 SDS(N)=SR(I,4)*FLUX(I,N) D 182
 0157 GO TO 54 D 183
 0158 53 SUM(N)=SUM(N)+FS(N) D 184
 0159 54 GO TO (56,56,55), I D 185
 0160 55 IF (N.GT.NINT(3)) GO TO 57 D 186
 0161 S(N)=SUM(N) D 187
 0162 SUM(N)=0.0 D 188
 0163 GO TO 58 D 189
 0164 56 S(N)=SDS(N) D 190
 0165 GO TO 58 D 191
 0166 57 S(N)=0.0 D 192
 0167 58 CONTINUE D 193
 C D 194
 C INTEGRATION OF THE SOURCE D 195
 C D 196
 0168 FMULT=S(N3)*RR(4)*H(4)/3. D 197
 0169 DO 63 K=1,3 D 198
 0170 GO TO (59,60,60), K D 199
 0171 59 L=0 D 200
 0172 GO TO 61 D 201
 0173 60 L=NINT(K-1)-1 D 202
 0174 61 M=NINT(K)-L D 203
 0175 R=RR(K) D 204
 0176 DO 62 N=L,M D 205
 0177 ITEMP=N+L D 206
 0178 GX(N)=S(ITEMP)*R D 207
 0179 62 R=R+H(K) D 208
 0180 63 FMULT=FMULT+SIMP(H,GX,M,K) D 209
 0181 FMULT=FMULT*2.*PI D 210
 0182 WRITE (6,159) FMULT D 211
 0183 IF (ABS(FMULT-FMANT)/FMULT-PREC/5.) 85,64,64 D 212
 C D 213
 C PREPARATION OF ITERATIONS D 214
 C D 215
 0184 64 RPSP=0.0 D 216
 0185 DO 66 N=1,N3 D 217
 0186 S(N)=S(N)/FMULT D 218
 0187 EMAX=ABS((S(N)-SANT(N))/SANT(N)) D 219
 0188 IF (EMAX-RPSP) 66,66,65 D 220
 0189 65 RPSP=EMAX D 221
 0190 66 CONTINUE D 222
 0191 EPS(JJ)=RPSP D 223
 0192 IF (JJ-20) 72,67,67 D 224
 0193 67 IF (ABS(EPS(10)/EPS(20))-1.5) 68,71,71 D 225
 0194 68 IF (ABS(EPS(20)/EPS(19)*(1.+ABS(EPS(19))))-1.) 71,69,69 D 226
 0195 69 GO TO (70,71), ITER D 227
 0196 70 OVREL=OVREL-0.1 D 228
 0197 71 JJ=1 D 229
 0198 72 DO 84 N=1,N3 D 230
 0199 Q=S(N)/SANT(N)-1. D 231
 0200 IF (Q) 73,74,74 D 232

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0201      73   K1=1
0202          GO TO 75
0203      74   K1=2
0204      75   Q=ABS(Q)*CVREL
0205          IF (Q-0.1) 77,76,76
0206      76   Q=1.0/EXP(Q)
0207          GO TO 80
0208      77   IF (Q-.001) 79,78,78
0209      78   Q=(1.-Q/2.)/(1.+Q/2.)
0210          GO TO 80
0211      79   Q=1.-Q
0212      80   GO TO (81,82), K1
0213      81   S(N)=SANT(N)*Q
0214          GC TO 83
0215      82   S(N)=SANT(N)*(2.-Q)
0216      83   SANT(N)=S(N)
0217      84   CONTINUE
0218          FMANT=FMULT
0219          JJ=JJ+1
0220          ITC=[ITC+1]
0221          IF (ITC.GT.400) GO TO 140
0222          GO TO 22
0223      85   GO TO (89,86), ITER
0224      86   DFMM=0.0
0225          DO 68 I=1,3
0226          IF (DFMAX(I)-DFMM) 88,88,87
0227      87   DFMM=DFMAX(I)
0228      88   CCNTINUE
0229          IF (DFMM-CONV) 89,64,64
0230      89   FME=1.
0231          DFMUL=FMULT-FME
0232          WRITE (6,160) XP,FMULT
0233          IF (XP.EQ.1.0) EIGEN=FMULT
0234          IF (ABS(DFMUL)-PREC) 104,90,90
0235      90   GO TO (91,92,99,103), NIT
0236      91   DFM1=DFMUL
0237          NIT=2
0238          GO TO 93
0239      92   IF (DFMUL*DFM1) 95,104,93
0240      93   IF (DFMUL) 94,104,94
0241      94   X1=XP
0242          XP=XP+PP*DFMUL/ABS(DFMUL)
0243          DFM1=DFMUL
0244          GO TO 9
0245      95   IF (DFMUL) 96,104,97
0246      96   DF2=DFM1
0247          DF1=DFMUL
0248          A20=X1
0249          A10=XP
0250          XP=X1-DFM1*(X1-XP)/(DFM1-DFMUL)
0251          GO TO 98
0252      97   DF2=DFMUL
0253          DF1=DFM1
0254          A20=XP
0255          A10=X1
0256          XP=XP-DFMUL*(XP-X1)/(DFMUL-DFM1)
0257      98   NIT=3
0258          X1=XP

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0259      DFM1=DFMUL          D 290
0260      GO TO 9            D 291
0261      99    NIT=4          D 292
0262      DFM1=DFMUL          D 293
0263      X1=XP              D 294
0264      IF (DFMUL) 100,104,101 D 295
0265      100    XP=X1-DFM1*(A20-X1)/(DF2-DFM1) D 296
0266      GO TO 102           D 297
0267      101    XP=X1-DFM1*(X1-A10)/(DFM1-DF1) D 298
0268      102    XANT=XP          D 299
0269      GO TO 9            D 300
0270      103    CALL ZERO (X1,XANT,DFM1,DFMUL,A10,A20,DF1,DF2,XP) D 301
0271      X1=XANT           D 302
0272      XANT=X1           D 303
0273      DFM1=DFMUL          D 304
0274      GO TO 9            D 305
0275      104    GO TO (105,106), ITER D 306
0276      105    ITER=2          D 307
0277      WRITE (6,161) ITER D 308
0278      GO TO 64             D 309
C
C      PREPARATION OF OUTPUT          D 310
C      CALCULATION OF POWER NORMALIZED FLUXES D 311
C
0279      106    FP=0.0          D 312
0280      DO 107 N=1,N3          D 313
0281      107    SUM(N)=0.0        D 314
0282      K=1                  D 315
0283      DO 110 I=1,3          D 316
0284      DO 110 N=1,N3M1        D 317
0285      IF (N-NINT(K)) 108,109,108 D 318
0286      108    FS(N)=XKF(I,K)*FLUX(I,N) D 319
0287      GO TO 110           D 320
0288      109    K=K+1          D 321
0289      TEMP=XKF(I,K-1)*FLUX(I,N) D 322
0290      FS(N)=XKF(I,K)*FLUX(I,N) D 323
0291      FS(N)=(FS(N)*H(K)+TEMP*H(K-1))/(H(K)+H(K-1)) D 324
0292      110    SUM(N)=SUM(N)+FS(N) D 325
0293      TEMP=H(3)/(H(3)+H(4)) D 326
0294      DO 111 I=1,3          D 327
0295      111    SUM(N3)=SUM(N3)+XKF(I,3)*FLUX(I,N3)*TEMP D 328
0296      DO 119 K=1,3          D 329
0297      GO TO (112,113,113), K D 330
0298      112    L=0              D 331
0299      GO TO 114           D 332
0300      113    L=NINT(K)-1 D 333
0301      114    M=NINT(K)-L D 334
0302      R=RR(K)           D 335
0303      DO 115 N=1,M          D 336
0304      ITEMP=N+L          D 337
0305      GX(N)=SUM(ITEMP)*R D 338
0306      115    R=R+H(K)        D 339
0307      FP=FP+SIMP(H,GX,M,K) D 340
0308      IF (K.EQ.3) FP=FP+SUM(N3)*RR(4)*H(4)/3. D 341
0309      GO TO (116,117,118), K D 342
0310      116    FPR(K)=FP        D 343
0311      GO TO 119           D 344
0312      117    FPR(K)=(FP-FPR(K-1)) D 345
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0313      GO TO 119
0314      118 FPR(K)=(FP-FPR(K-1)-FPR(K-2))
0315      119 CONTINUE
0316      FP=0.0
0317      DO 120 K=1,3
0318      FPR(K)=2.*PI*FPR(K)
0319      120 FP=FP+FPR(K)
0320      GAM=POW/FP
0321      TEMP=PI/SQRT(BH2)
0322      GAM=GAM/TEMP
0323      WRITE (6,162)
0324      WRITE (6,163) (RR(L),L=1,5)
0325      WRITE (6,164) EIGEN
0326      WRITE (6,165)
0327      GO TO (121,121,121,122,123,124), ICP
0328      121 WRITE (6,166) ICP
0329      GO TO 125
0330      122 WRITE (6,167)
0331      GO TO 125
0332      123 WRITE (6,168)
0333      GO TO 125
0334      124 WRITE (6,169)
0335      125 DO 126 K=1,3
0336      126 SCA(3,K)=SCA(3,K)-D(3,K)*BH2
0337      GO TO (127,128,129,127,127,128), ICP
0338      127 K=1
0339      GO TO 130
0340      128 K=2
0341      GO TO 130
0342      129 K=3
0343      130 TEMP=SCA(3,K)-SCAO(3,K)
0344      WRITE (6,170) TEMP
0345      TEMP=TEMP/SCP*1.E+24
0346      WRITE (6,171) TEMP
0347      WRITE (6,172)
0348      WRITE (6,173) (FLUX(1,N),N=1,N4)
0349      WRITE (6,174) (FLUX(2,N),N=1,N4)
0350      WRITE (6,175) (FLUX(3,N),N=1,N4)

C
C      CALCULATION OF POWER NORMALIZED FLUXES
C
0351      DO 131 I=1,3
0352      DO 131 N=1,N4
0353      FLUX(I,N)=FLUX(I,N)*GAM
0354      131 AIDD(I,N)=FLUX(I,N)
0355      WRITE (6,176)
0356      WRITE (6,177) (AIDD(1,N),N=1,N4)
0357      WRITE (6,178) (AIDD(2,N),N=1,N4)
0358      WRITE (6,179) (AIDD(3,N),N=1,N4)
0359      DO 132 LI=1,3
0360      LP=LI
0361      CALL PLOT (LI,N4,LP)
0362      132 CONTINUE
0363      WRITE (6,180)
0364      DO 133 I=1,3
0365      TEMP=FLUX(I,1)
0366      DO 133 N=1,N4
0367      133 AIDD(I,N)=FLUX(I,N)/TEMP

```

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0368      WRITE (6,181) (AIDD(1,N),N=1,N4)          D 406
0369      WRITE (6,182) (AIDD(2,N),N=1,N4)          D 407
0370      WRITE (6,183) (AIDD(3,N),N=1,N4)          D 408
0371      DO 134 LI=1,3                           D 409
0372      LP=LI+3                                D 410
0373      CALL PLOT (LI,N4,LP)                      D 411
0374      134  CONTINUE                            D 412
0375      DO 135 I=1,2                           D 413
0376      DO 135 N=1,N4                         D 414
0377      135  AIDD(I,N)=FLUX(I,N)/FLUX(3,N)    D 415
0378      WRITE (6,184)                           D 416
0379      WRITE (6,185) (AIDD(1,N),N=1,N4)          D 417
0380      WRITE (6,186) (AIDD(2,N),N=1,N4)          D 418
0381      DO 136 LI=1,2                           D 419
0382      LP=6+LI                                D 420
0383      CALL PLOT (LI,N4,LP)                      D 421
0384      136  CONTINUE                            D 422
0385      DO 137 N=1,N4                         D 423
0386      137  AIDD(1,N)=FLUX(1,N)/FLUX(2,N)    D 424
0387      WRITE (6,187) (AIDD(1,N),N=1,N4)          D 425
0388      CALL PLOT (1,N4,9)                        D 426
C
C      CALCULATION OF POWER GENERATED IN EACH REGION   D 427
C
0389      DO 138 K=1,3                           D 428
0390      TEMP=PI/SQRT(BH2)                      D 429
0391      FPR(K)=FPR(K)*TEMP*GAM                D 430
0392      138  CONTINUE                            D 431
0393      WRITE (6,188) (FPR(K),K=1,3)            D 432
0394      DO 139 K=1,3                           D 433
0395      FPR(K)=FPR(K)/TEMP                    D 434
0396      TEMP=PI*(RR(K+1)*RR(K+1)-RR(K)*RR(K)) D 435
0397      FPR(K)=FPR(K)/TEMP                    D 436
0398      139  CONTINUE                            D 437
0399      WRITE (6,189) (FPR(K),K=1,3)            D 438
0400      WRITE (6,190) OVREL                   D 439
0401      IF (ICC.GT.0) GC TO 1                  D 440
0402      GO TO 141                             D 441
0403      140  WRITE (6,191)                      D 442
0404      141  STCP                               D 443
C
0405      142  FORMAT (6E10.0,6(1X,I2))          41
0406      143  FORMAT (5E10.0)
0407      144  FORMAT (3E10.0)
0408      145  FORMAT (4E10.0)
0409      146  FORMAT (F5.3)
0410      147  FORMAT (1H1)
0411      148  FORMAT (16H PROGRAM WAN DIC,///,26H REGIONS IN DRIVER 1,2,3,4,//)
0412      149  FORMAT (17H RADII OF REGIONS,/,2X,8HREGION 1,4X 3HREGION 2,4X,8H
0413      150  1EGION 3,4X,8HREGION 4,4X,8HREGION 5,/,2X,5(F8.2,4X))
0414      151  FORMAT (7H NINT =,4(1Z,1X),/,7H OVREL=,F6.2,6X,6H PREC=,E9.2,6X,6H
0415      152  1 CONV=,E9.2)
0416      153  FORMAT (//,6H BH2 =,E12.5,6X,28H POISON MICRO CROSS-SECTION=,1X,E1
0417      154  12.5,/)
0418      155  FORMAT (26H ABSORPTION CROSS-SECTIONS,/,5X,7HGROUP 1,7X,7HGROUP 2
0419      156  1,7X,7HGROUP 3,/,3(2X,E12.5),/)
0420      157  FORMAT (//,23H DIFFUSION COEFFICIENTS,/,3(2X,E12.5),/)
0421      158  FORMAT (//,26H TRANSFER CROSS-SECTIONS ,/,3(2X,E12.5),/)
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0418	155	FORMAT (//,23H NU-FISSION ,//,3(2X,E12.5),/)	D 464
0419	156	FORMAT (//,14H KAPPA-FISSION,/,3(2X,E12.5),/)	D 465
0420	157	FORMAT (13H POWER (W) = ,E12.5)	D 466
0421	158	FORMAT (1H1)	D 467
0422	159	FCRMAT (1X,4H K=,F10.6)	D 468
0423	160	FORMAT (//,5H XP= ,E12.5,5X,15H EIGENVALUE K =,E12.5)	D 469
0424	161	FORMAT (7H ITER= ,I2)	D 470
0425	162	FORMAT (1H1,8H RESULTS,///)	D 471
0426	163	FORMAT (17H RADII OF REGIONS,/,2X,8HREGION 1,4X,8HREGION 2,4X,8H REGION 3,4X,8HREGION 4,4X,8HREGION 5,/,2X,5(F8.2,4X))	D 472
0427	164	FORMAT (//,41H EIGENVALUE OF THE UNPOISONED REACTOR K =,E12.5)	D 473
0428	165	FORMAT (//,49H CONTROL POISON WAS USED IN THE FOLLOWING REGIONS,/) D 474	
0429	166	FORMAT (9H REGION ,I2)	D 475
0430	167	FCRMAT (21H ALL FISSION REGIONS)	D 476
0431	168	FORMAT (17H REGIONS 1 AND 2)	D 477
0432	169	FORMAT (17H REGIONS 2 AND 3)	D 478
0433	170	FORMAT (//,36H MACRO CROSS-SECTION OF THE POISON= ,E12.5)	D 479
0434	171	FORMAT (//,24H POISON CONCENTRATION = ,E12.5)	D 480
0435	172	FORMAT (//,28H FLUXES NORMALIZED TO SOURCE)	D 481
0436	173	FORMAT (//,19H FAST FLUX-GROUP 1 ,/,5(2X,E12.5))	D 482
0437	174	FORMAT (//,24H RESONANCE FLUX-GROUP 2 ,/,5(2X,E12.5))	D 483
0438	175	FORMAT (//,19H SLOW FLUX-GROUP 3 ,/,5(2X,E12.5))	D 484
0439	176	FORMAT (//,28H FLUXES NORMALIZED TO PCWER)	D 485
0440	177	FORMAT (//,19H FAST FLUX-GROUP 1 ,/,5(2X,E12.5))	D 486
0441	178	FORMAT (//,24H RESONANCE FLUX-GROUP 2 ,/,5(2X,E12.5))	D 487
0442	179	FORMAT (//,19H SLOW FLUX-GROUP 3 ,/,5(2X,E12.5))	D 488
0443	180	FORMAT (//,16H RELATIVE FLUXES)	D 489
0444	181	FORMAT (//,9H GROUP 1 ,/.5(2X,E12.5))	D 490
0445	182	FORMAT (//,9H GROUP 2 ,/.5(2X,E12.5))	D 491
0446	183	FORMAT (//,9H GROUP 3 ,/.5(2X,E12.5))	D 492
0447	184	FORMAT (//,35H RATIOS OF FLUXES TO THE SLOW FLUX)	D 493
0448	185	FORMAT (//,19H GROUP 1 / GROUP 3 ,/.5(2X,E12.5))	D 494
0449	186	FORMAT (//,19H GROUP 2 / GROUP 3 ,/.5(2X,E12.5))	D 495
0450	187	FORMAT (//,19H GROUP 1 / GROUP 2 ,//,5(2X,E12.5))	D 496
0451	188	FORMAT (//,31H POWER OUTPUT IN REGIONS 1,2,3,/,2X,3(E12.5,5X))	D 497
0452	189	FORMAT (//,42H POWER DENSITY IN REGIONS 1,2,3 (KW/LITER),/,2X,3(E 12.5,5X))	D 498
0453	190	FORMAT (//,33H COMPUTED OVERRELAXATION FACTOR= ,F6.3)	D 499
0454	191	FORMAT (22H # OF ITERATIONS > 400)	D 500
0455		END	D 501

PROGRAM WAN DIC

REGIONS IN ORDER 1,2,3,4,

RADIIS OF REGIONS

REGION 1	REGION 2	REGION 3	REGION 4	REGION 5
0.0	87.70	124.00	152.00	159.00
NINT = 30 20 20 4				
OVREL= 1.90	PREC= 0.10E-03		CONV= 0.10E-01	

BH2 = 0.73800E-04 POISON MICRO CROSS-SECTION= 0.21090E 04

ABSORPTION CROSS-SECTIONS

GROUP 1	GROUP 2	GROUP 3
0.35410E-02	0.25680E-01	0.13870E 00
0.35410E-02	0.25680E-01	0.13870E 00
0.35410E-02	0.25680E-01	0.13870E 00
0.36220E-03	0.57220E-03	0.84730E-02

DIFFUSION COEFFICIENTS

0.17070E 01	0.85360E 00	0.39510E 00
0.17070E 01	0.85360E 00	0.39510E 00
0.17070E 01	0.85360E 00	0.39510E 00
0.19300E 01	0.91730E 00	0.31470E 00

TRANSFER CROSS-SECTIONS

0.61130E-01	0.49440E-01	0.0
0.61130E-01	0.49440E-01	0.0
0.61130E-01	0.49440E-01	0.0
0.26280E-01	0.28110E-01	0.0

NU-FISSION

0.42920E-02	0.15060E-01	0.16400E 00
0.42920E-02	0.15060E-01	0.16400E 00
0.42920E-02	0.15060E-01	0.16400E 00

KAPPA-FISSION

0.93280E-13	0.19010E-12	0.21290E-11
0.93280E-13	0.19010E-12	0.21290E-11
0.93280E-13	0.19010E-12	0.21290E-11

POWER {W} = 0.24410E 10

RESULTS

RAOII OF REGIONS

REGION 1	REGION 2	REGION 3	REGION 4	REGION 5
0.0	87.70	124.00	152.00	159.00

EIGENVALUE OF THE UNPOISONED REACTOR K = 0.97971E 00

CONTROL POISON WAS USED IN THE FOLLOWING REGIONS

ALL FISSION REGIONS

MACRO CROSS-SECTION OF THE POISON= -0.37987E-02

POISON CONCENTRATION = -0.18012E 19

FLUXES NORMALIZED TO SOURCE

FAST FLUX-GROUP 1

0.42922E-03	0.42904E-03	0.42848E-03	0.42755E-03	0.42625E-03
0.42457E-03	0.42253E-03	0.42012E-03	0.41734E-03	0.41420E-03
0.41070E-03	0.40683E-03	0.40261E-03	0.39804E-03	0.39312E-03
0.38785E-03	0.38224E-03	0.37629E-03	0.37002E-03	0.36342E-03
0.35650E-03	0.34927E-03	0.34173E-03	0.33390E-03	0.32578E-03
0.31737E-03	0.30870E-03	0.29977E-03	0.29059E-03	0.28117E-03
0.27153E-03	0.26543E-03	0.25925E-03	0.25300E-03	0.24667E-03
0.24027E-03	0.23381E-03	0.22728E-03	0.22069E-03	0.21404E-03
0.20734E-03	0.20059E-03	0.19379E-03	0.18696E-03	0.18008E-03
0.17317E-03	0.16623E-03	0.15926E-03	0.15228E-03	0.14527E-03
0.13825E-03	0.13282E-03	0.12740E-03	0.12197E-03	0.11654E-03
0.11111E-03	0.10569E-03	0.10027E-03	0.94856E-04	0.89454E-04
0.84063E-04	0.78686E-04	0.73325E-04	0.67980E-04	0.62653E-04
0.57341E-04	0.52038E-04	0.46722E-04	0.41343E-04	0.35784E-04
0.29774E-04	0.23213E-04	0.17707E-04	0.13012E-04	0.89179E-05

RESONANCE FLUX-GROUP 2

0.34819E-03	0.34804E-03	0.34759E-03	0.34683E-03	0.34577E-03
0.34441E-03	0.34275E-03	0.34080E-03	0.33854E-03	0.33599E-03
0.33314E-03	0.33001E-03	0.32658E-03	0.32287E-03	0.31887E-03
0.31459E-03	0.31004E-03	0.30521E-03	0.30012E-03	0.29476E-03
0.28914E-03	0.28327E-03	0.27716E-03	0.27080E-03	0.26421E-03
0.25739E-03	0.25036E-03	0.24311E-03	0.23566E-03	0.22802E-03
0.22019E-03	0.21525E-03	0.21023E-03	0.20516E-03	0.20003E-03
0.19484E-03	0.18959E-03	0.18429E-03	0.17895E-03	0.17356E-03
0.16812E-03	0.16265E-03	0.15714E-03	0.15159E-03	0.14602E-03
0.14041E-03	0.13478E-03	0.12913E-03	0.12347E-03	0.11778E-03
0.11209E-03	0.10769E-03	0.10329E-03	0.98890E-04	0.94487E-04
0.90086E-04	0.85687E-04	0.81293E-04	0.76904E-04	0.72522E-04
0.68149E-04	0.63786E-04	0.59433E-04	0.55091E-04	0.50761E-04
0.46441E-04	0.42132E-04	0.37833E-04	0.33548E-04	0.29291E-04
0.25108E-04	0.20280E-04	0.15418E-04	0.10537E-04	0.55911E-05

SLOW FLUX-GROUP 3

0.12748E-03	0.12742E-03	0.12726E-03	0.12698E-03	0.12659E-03
0.12609E-03	0.12549E-03	0.12477E-03	0.12394E-03	0.12301E-03
0.12197E-03	0.12082E-03	0.11956E-03	0.11820E-03	0.11674E-03
0.11517E-03	0.11351E-03	0.11174E-03	0.10987E-03	0.10791E-03
0.10586E-03	0.10371E-03	0.10147E-03	0.99139E-04	0.96725E-04
0.94229E-04	0.91653E-04	0.89000E-04	0.86272E-04	0.83474E-04
0.806C9E-04	0.78797E-04	0.76962E-04	0.75105E-04	0.73225E-04
0.71325E-04	0.69405E-04	0.67466E-04	0.65509E-04	0.63535E-04
0.61545E-04	0.59541E-04	0.57524E-04	0.55494E-04	0.53452E-04
0.51401E-04	0.49340E-04	0.47272E-04	0.45197E-04	0.43117E-04
0.41033E-04	0.39423E-04	0.37812E-04	0.36200E-04	0.34588E-04
0.32977E-04	0.31367E-04	0.29758E-04	0.28152E-04	0.26548E-04
0.24948E-04	0.23353E-04	0.21764E-04	0.20186E-04	0.18625E-04
0.17100E-04	0.15647E-04	0.14350E-04	0.13391E-04	0.13176E-04
0.14593E-04	0.16797E-04	0.14841E-04	0.99331E-05	0.30294E-05

FLUXES NORMALIZED TO POWER

FAST FLUX-GROUP 1

0.21237E 15	0.21228E 15	0.21200E 15	0.21154E 15	0.21090E 15
0.21007E 15	0.20906E 15	0.20787E 15	0.20649E 15	0.20494E 15
0.20320E 15	0.20129E 15	0.19920E 15	0.19694E 15	0.19451E 15
0.19190E 15	0.18912E 15	0.18618E 15	0.18308E 15	0.17981E 15
0.17639E 15	0.17281E 15	0.16908E 15	0.16520E 15	0.16119E 15
0.15703E 15	0.15274E 15	0.14832E 15	0.14378E 15	0.13912E 15
0.13435E 15	0.13133E 15	0.12827E 15	0.12518E 15	0.12205E 15
0.11888E 15	0.11568E 15	0.11245E 15	0.10919E 15	0.10590E 15
0.10259E 15	0.99247E 14	0.95885E 14	0.92502E 14	0.89100E 14
0.85681E 14	0.82248E 14	0.78801E 14	0.75343E 14	0.71876E 14
0.68402E 14	0.65719E 14	0.63034E 14	0.60347E 14	0.57661E 14
0.54975E 14	0.52291E 14	0.49610E 14	0.46933E 14	0.44260E 14
0.41593E 14	0.38932E 14	0.36279E 14	0.33635E 14	0.30999E 14
0.28371E 14	0.25747E 14	0.23117E 14	0.20456E 14	0.17705E 14
0.14731E 14	0.11485E 14	0.87612E 13	0.64379E 13	0.44124E 13

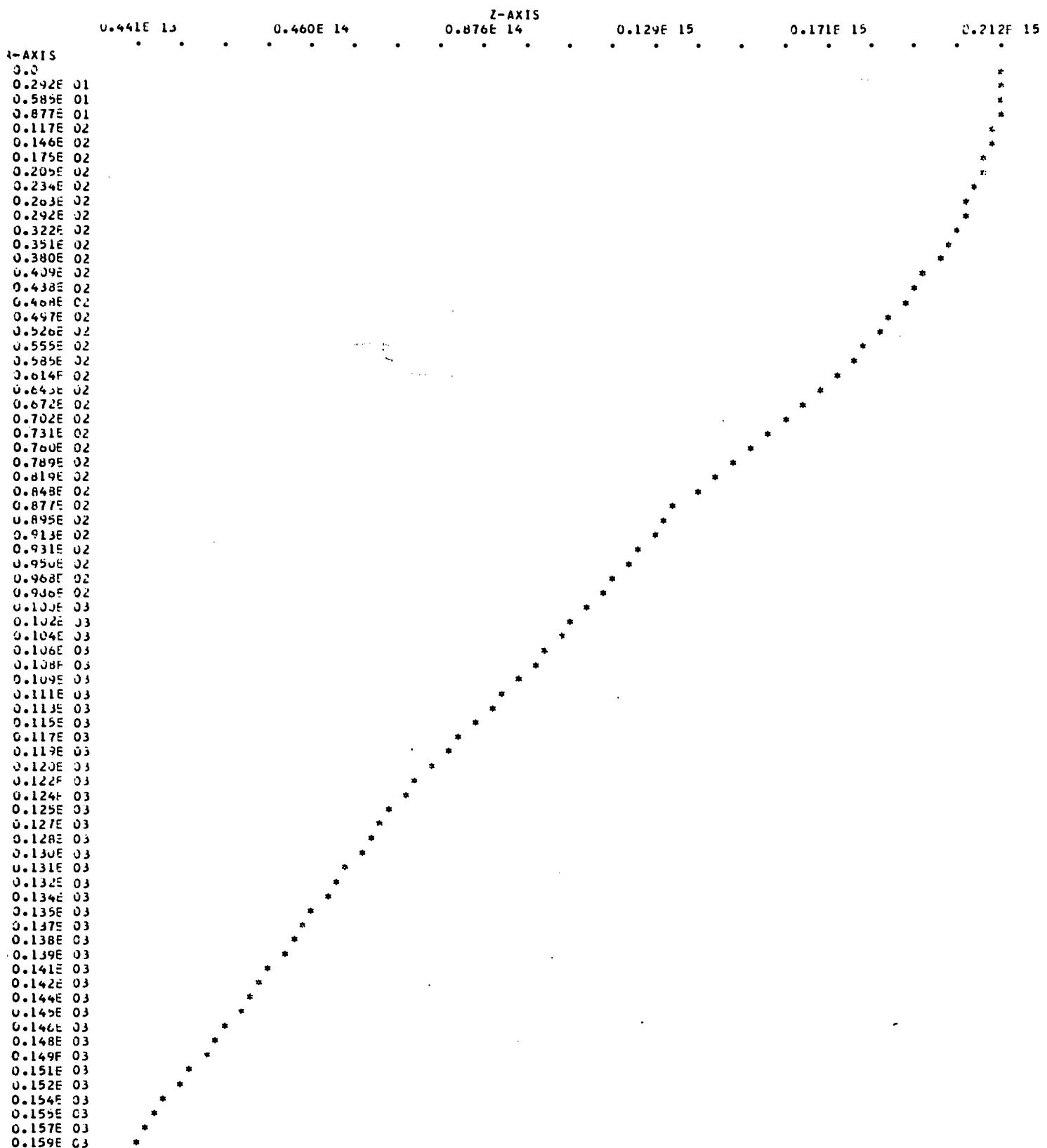
RESONANCE FLUX-GROUP 2

0.17228E 15	0.17220E 15	0.17198E 15	0.17160E 15	0.17108E 15
0.17041E 15	0.16959E 15	0.16862E 15	0.16750E 15	0.16624E 15
0.16483E 15	0.16328E 15	0.16159E 15	0.15975E 15	0.15777E 15
0.15565E 15	0.15340E 15	0.15101E 15	0.14849E 15	0.14584E 15
0.14306E 15	0.14016E 15	0.13713E 15	0.13399E 15	0.13073E 15
0.12735E 15	0.12387E 15	0.12029E 15	0.11660E 15	0.11282E 15
0.10895E 15	0.10650E 15	0.10402E 15	0.10151E 15	0.98969E 14
0.96400E 14	0.93805E 14	0.91185E 14	0.88540E 14	0.85872E 14
0.83184E 14	0.80475E 14	0.77748E 14	0.75005E 14	0.72246E 14
0.69473E 14	0.66688E 14	0.63893E 14	0.61089E 14	0.58277E 14
0.55460E 14	0.53284E 14	0.51107E 14	0.48929E 14	0.46750E 14
0.44572E 14	0.42396E 14	0.40222E 14	0.38050E 14	0.35882E 14
0.33719E 14	0.31560E 14	0.29406E 14	0.27258E 14	0.25115E 14
0.22978E 14	0.20846E 14	0.18719E 14	0.16599E 14	0.14433E 14
0.12423E 14	0.10034E 14	0.76287E 13	0.52137E 13	0.276...E 13

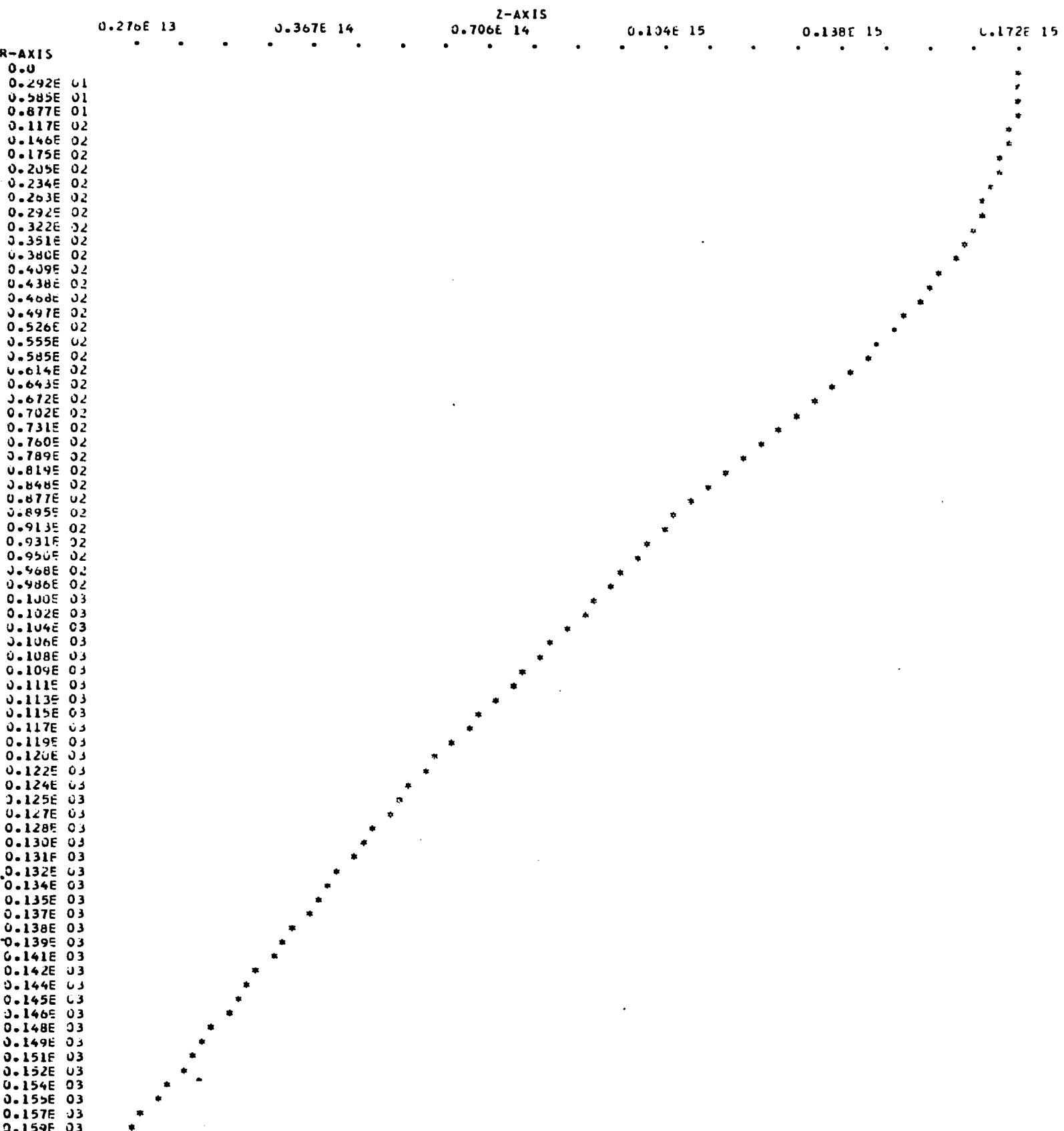
SLOW FLUX-GROUP 3

0.63073E 14	0.63046E 14	0.62963E 14	0.62826E 14	0.62635E 14
0.62388E 14	0.62088E 14	0.61733E 14	0.61324E 14	0.60862E 14
0.60347E 14	0.59778E 14	0.59158E 14	0.58485E 14	0.57761E 14
0.56986E 14	0.56161E 14	0.55286E 14	0.54363E 14	0.53392E 14
0.52375E 14	0.51311E 14	0.50203E 14	0.49052E 14	0.47858E 14
0.46623E 14	0.45348E 14	0.44035E 14	0.42686E 14	0.41301E 14
0.39883E 14	0.38987E 14	0.38079E 14	0.37160E 14	0.36230E 14
0.35290E 14	0.34340E 14	0.33380E 14	0.32412E 14	0.31436E 14
0.30451E 14	0.29460E 14	0.28461E 14	0.27457E 14	0.26447E 14
0.25432E 14	0.24412E 14	0.23389E 14	0.22363E 14	0.21333E 14
0.20302E 14	0.19506E 14	0.18708E 14	0.17911E 14	0.17114E 14
0.16316E 14	0.15520E 14	0.14724E 14	0.13929E 14	0.13135E 14
0.12344E 14	0.11554E 14	0.10768E 14	0.99874E 13	0.92153E 13
0.84606E 13	0.77418E 13	0.70999E 13	0.66258E 13	0.65193E 13
0.72204E 13	0.83110E 13	0.73431E 13	0.49147E 13	0.14989E 13

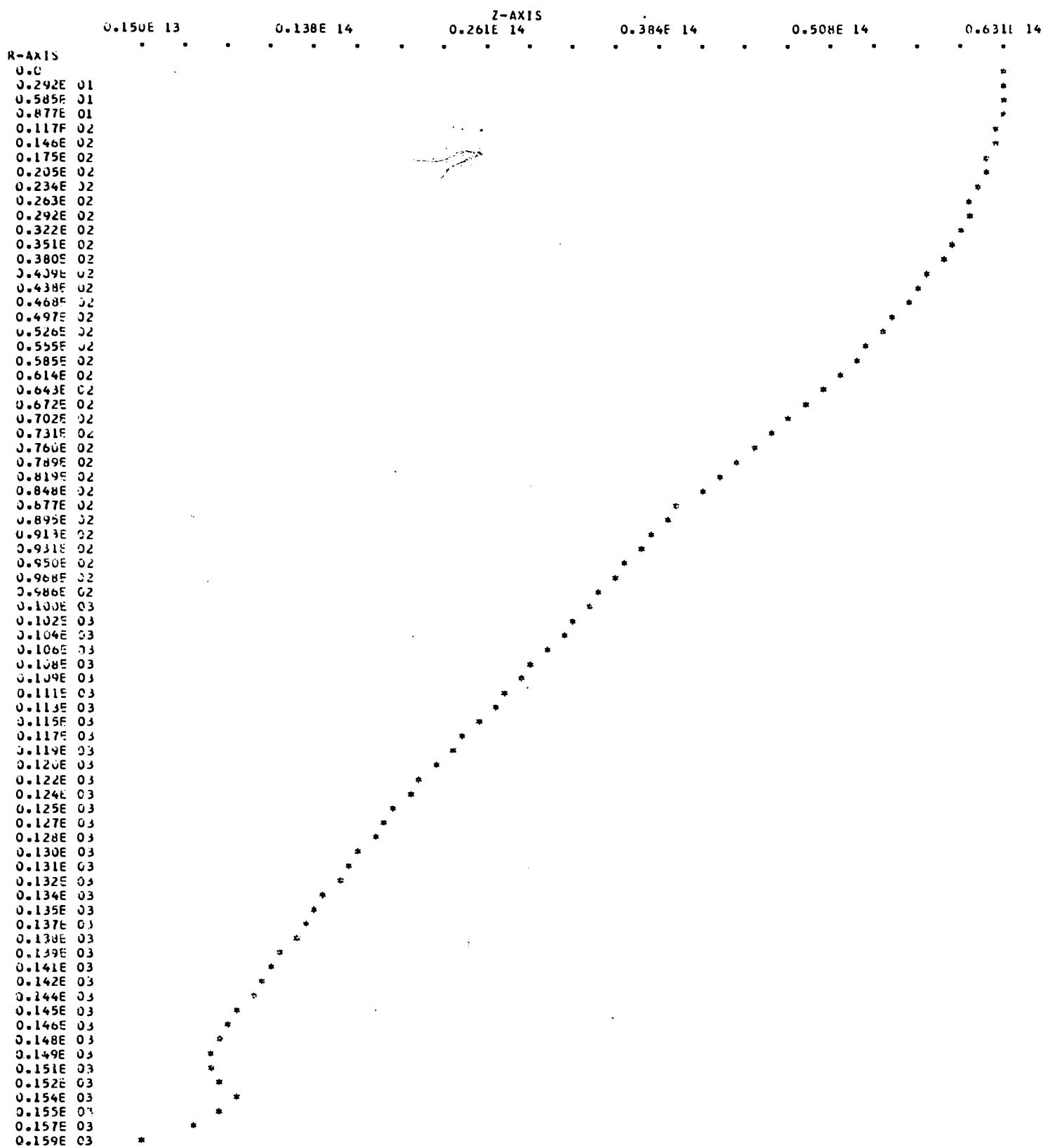
PLOT OF FAST FLUX - GROUP 1



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PLOT OF RESONANCE FLUX - GROUP 2



PLOT OF SLOW FLUX - GROUP 3



RELATIVE FLUXES

GROUP 1

0.10000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99306E 00
0.98916E 00	0.98440E 00	0.97879E 00	0.97232E 00	0.96500E 00
0.95683E 00	0.94783E 00	0.93800E 00	0.92735E 00	0.91588E 00
0.90360E 00	0.89053E 00	0.87668E 00	0.86206E 00	0.84668E 00
0.83056E 00	0.81371E 00	0.79616E 00	0.77791E 00	0.75899E 00
0.73942E 00	0.71921E 00	0.69841E 00	0.67702E 00	0.65507E 00
0.63260E 00	0.61839E 00	0.60400E 00	0.58943E 00	0.57469E 00
0.55978E 00	0.54472E 00	0.52950E 00	0.51415E 00	0.49866E 00
0.48305E 00	0.46733E 00	0.45150E 00	0.43557E 00	0.41955E 00
0.40345E 00	0.38728E 00	0.37105E 00	0.35477E 00	0.33845E 00
0.32209E 00	0.30945E 00	0.29681E 00	0.28416E 00	0.27151E 00
0.25886E 00	0.24623E 00	0.23360E 00	0.22099E 00	0.20841E 00
0.19585E 00	0.18332E 00	0.17083E 00	0.15838E 00	0.14597E 00
0.13359E 00	0.12124E 00	0.10885E 00	0.96321E-01	0.83369E-01
0.69366E-01	0.54081E-01	0.41254E-01	0.30314E-01	0.20777E-01

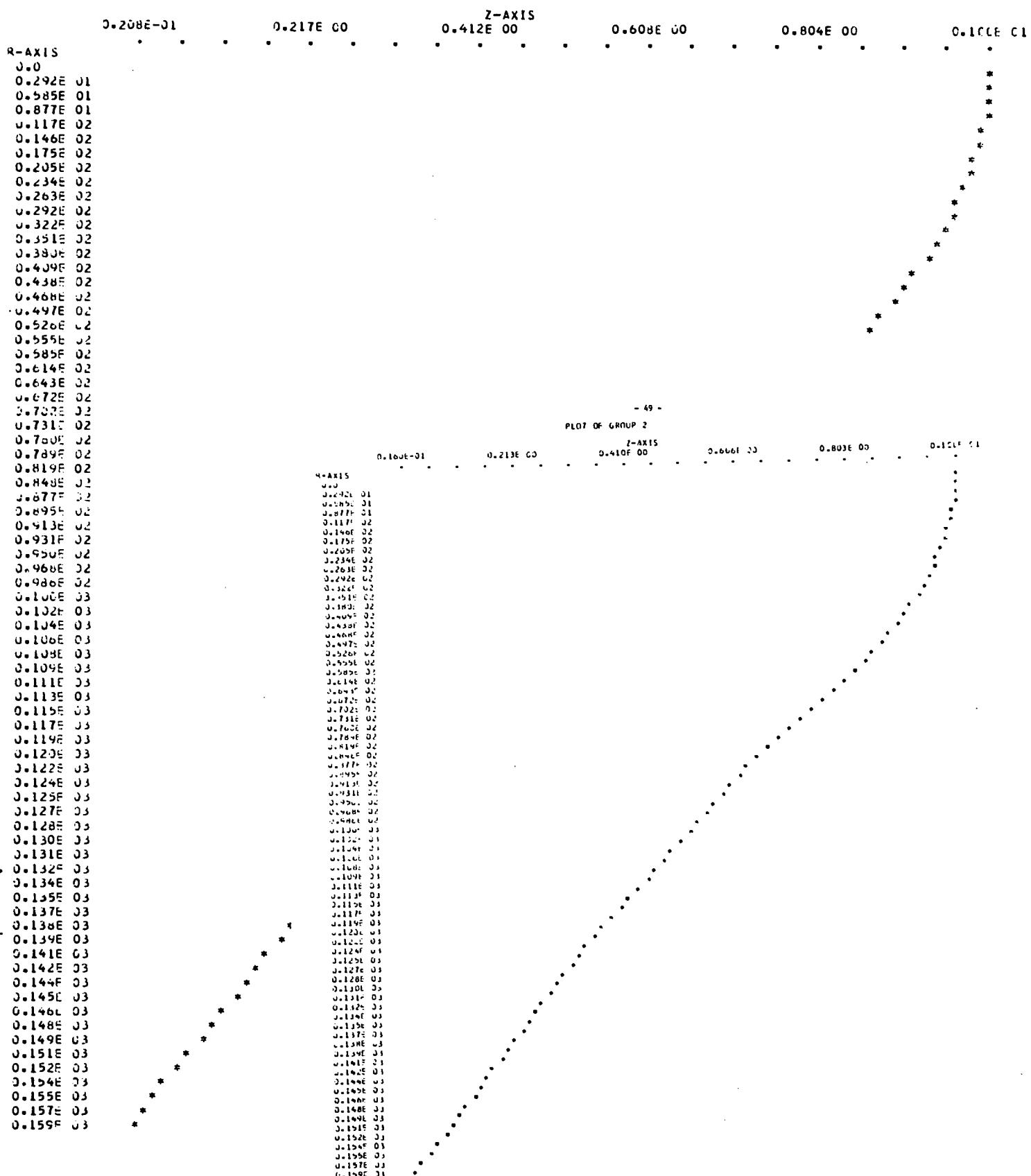
GROUP 2

0.10000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99305E 00
0.98915E 00	0.98439E 00	0.97876E 00	0.97229E 00	0.96496E 00
0.95679E 00	0.94778E 00	0.93794E 00	0.92727E 00	0.91580E 00
0.90351E 00	0.89043E 00	0.87657E 00	0.86194E 00	0.84655E 00
0.83042E 00	0.81356E 00	0.79600E 00	0.77774E 00	0.75881E 00
0.73923E 00	0.71902E 00	0.69821E 00	0.67682E 00	0.65487E 00
0.63239E 00	0.61818E 00	0.60379E 00	0.58922E 00	0.57447E 00
0.55957E 00	0.54450E 00	0.52929E 00	0.51394E 00	0.49846E 00
0.48285E 00	0.46713E 00	0.45130E 00	0.43537E 00	0.41936E 00
0.40326E 00	0.38710E 00	0.37087E 00	0.35460E 00	0.33828E 00
0.32192E 00	0.30930E 00	0.29666E 00	0.28401E 00	0.27137E 00
0.25873E 00	0.24609E 00	0.23347E 00	0.22087E 00	0.20828E 00
0.19572E 00	0.18319E 00	0.17069E 00	0.15822E 00	0.14578E 00
0.13338E 00	0.12100E 00	0.10866E 00	0.96349E-01	0.84124E-01
0.72109E-01	0.58245E-01	0.44281E-01	0.30263E-01	0.16029E-01

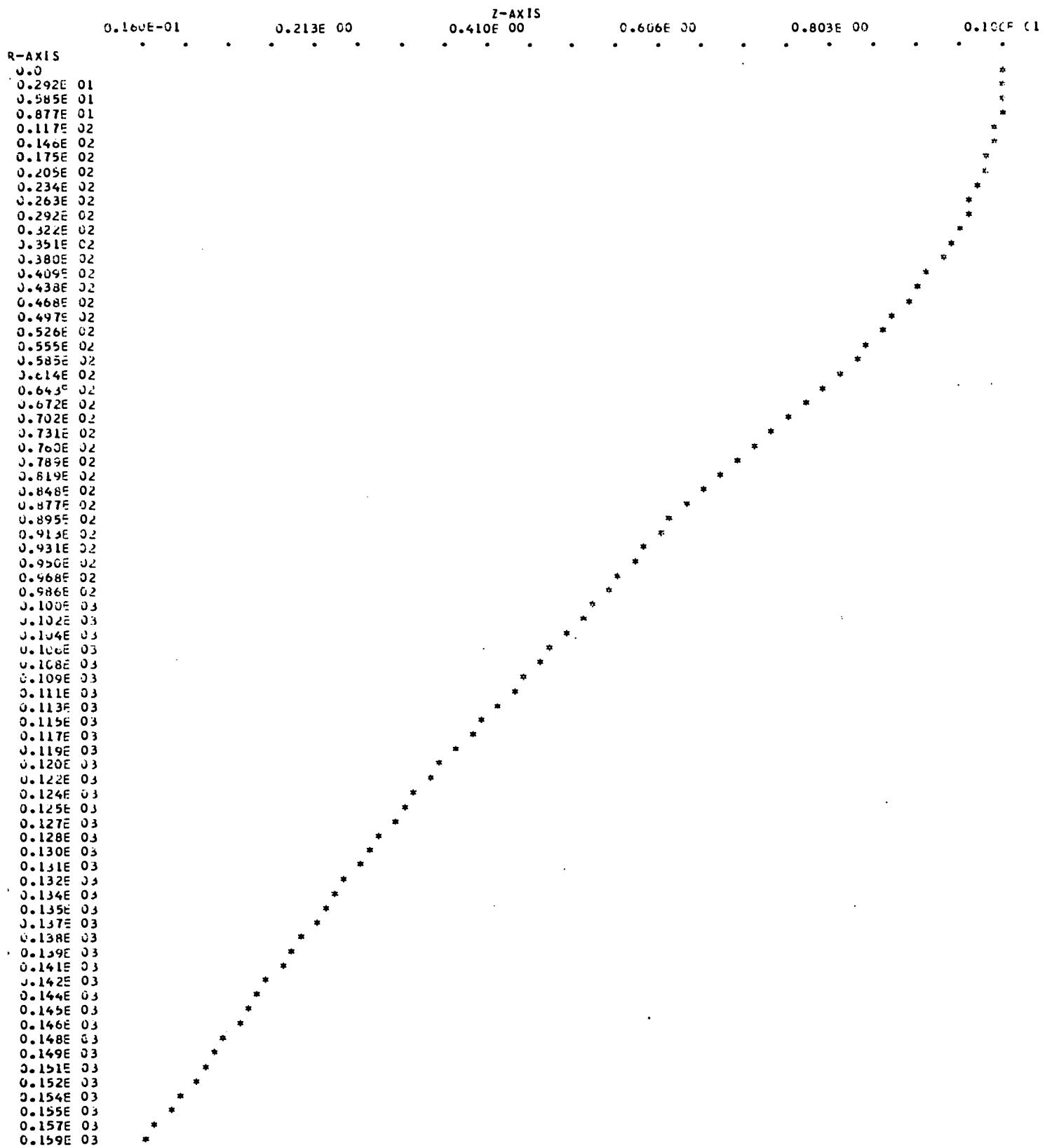
GROUP 3

0.10000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99305E 00
0.98915E 00	0.98438E 00	0.97876E 00	0.97228E 00	0.96495E 00
0.95678E 00	0.94776E 00	0.93792E 00	0.92725E 00	0.91577E 00
0.90349E 00	0.89041E 00	0.87654E 00	0.86191E 00	0.84652E 00
0.83038E 00	0.81352E 00	0.79596E 00	0.77770E 00	0.75877E 00
0.73918E 00	0.71897E 00	0.69816E 00	0.67677E 00	0.65482E 00
0.63234E 00	0.61813E 00	0.60373E 00	0.58916E 00	0.57442E 00
0.55951E 00	0.54445E 00	0.52924E 00	0.51388E 00	0.49840E 00
0.48279E 00	0.46707E 00	0.45125E 00	0.43532E 00	0.41931E 00
0.40321E 00	0.38705E 00	0.37083E 00	0.35455E 00	0.33823E 00
0.32188E 00	0.30925E 00	0.29662E 00	0.28397E 00	0.27133E 00
0.25869E 00	0.24606E 00	0.23344E 00	0.22084E 00	0.20826E 00
0.19571E 00	0.18319E 00	0.17073E 00	0.15835E 00	0.14611E 00
0.13414E 00	0.12274E 00	0.11257E 00	0.10505E 00	0.10336E 00
0.11448E 00	0.13177E 00	0.11642E 00	0.77920E-01	0.23764E-01

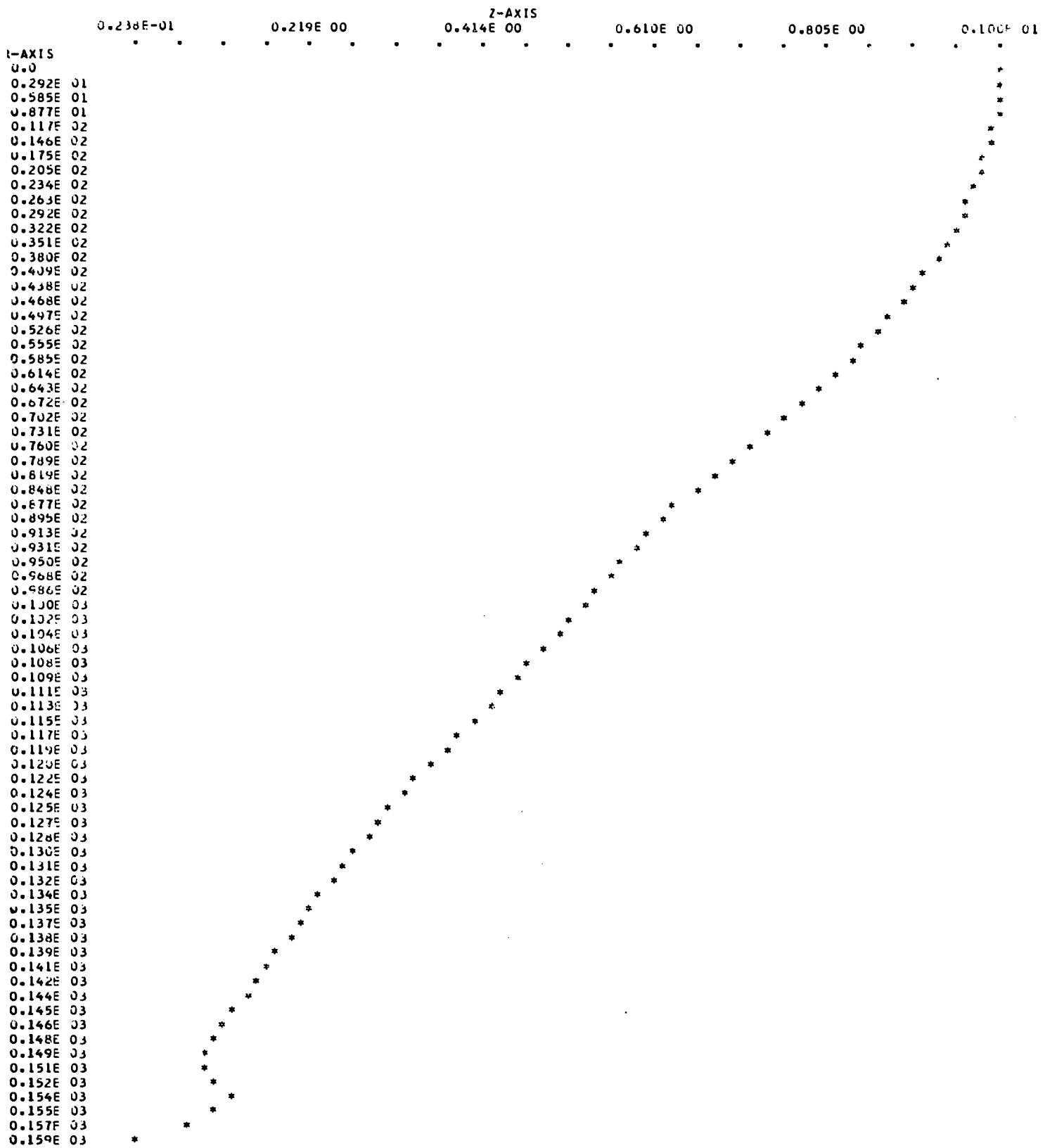
PLOT OF GROUP 1



PLOT OF GROUP 2



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PLOT OF GROUP 3



RATIOS OF FLUXES TO THE SLOW FLUX

GROUP 1 / GROUP 3

0.33671E 01				
0.33671E 01	0.33671E 01	0.33672E 01	0.33672E 01	0.33672E 01
0.33673E 01	0.33673E 01	0.33673E 01	0.33674E 01	0.33674E 01
0.33675E 01	0.33675E 01	0.33676E 01	0.33677E 01	0.33677E 01
0.33678E 01	0.33679E 01	0.33679E 01	0.33680E 01	0.33680E 01
0.33681E 01	0.33682E 01	0.33683E 01	0.33683E 01	0.33684E 01
0.33685E 01	0.33685E 01	0.33686E 01	0.33686E 01	0.33686E 01
0.33687E 01	0.33687E 01	0.33688E 01	0.33688E 01	0.33688E 01
0.33689E 01	0.33689E 01	0.33689E 01	0.33690E 01	0.33690E 01
0.33690E 01	0.33691E 01	0.33691E 01	0.33691E 01	0.33692E 01
0.33692E 01	0.33692E 01	0.33693E 01	0.33693E 01	0.33693E 01
0.33693E 01	0.33694E 01	0.33694E 01	0.33694E 01	0.33695E 01
0.33695E 01	0.33695E 01	0.33691E 01	0.33677E 01	0.33639E 01
0.33533E 01	0.33257E 01	0.32559E 01	0.30873E 01	0.27158E 01
0.20403E 01	0.13819E 01	0.11931E 01	0.13099E 01	0.29438E 01

GROUP 2 / GROUP 3

0.27314E 01				
0.27314E 01				
0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01	0.27315E 01
0.27315E 01				
0.27315E 01	0.27315E 01	0.27315E 01	0.27315E 01	0.27316E 01
0.27316E 01				
0.27316E 01	0.27316E 01	0.27316E 01	0.27316E 01	0.27317E 01
0.27317E 01				
0.27317E 01				
0.27317E 01				
0.27317E 01	0.27318E 01	0.27318E 01	0.27318E 01	0.27318E 01
0.27318E 01	0.27318E 01	0.27318E 01	0.27318E 01	0.27317E 01
0.27316E 01	0.27314E 01	0.27308E 01	0.27292E 01	0.27254E 01
0.27159E 01	0.26927E 01	0.26365E 01	0.25052E 01	0.22230E 01
0.17205E 01	0.12074E 01	0.10389E 01	0.10608E 01	0.18423E 01

PLOT OF GROUP 1 / GROUP 3

Z-AXIS

X-AXIS	0.119E 01	0.163E 01	0.206E 01	0.250E 01	0.293E 01	0.337E 01
0.0	*	*	*	*	*	*
0.292E 01	*	*	*	*	*	*
0.585E 01	*	*	*	*	*	*
0.677E 01	*	*	*	*	*	*
0.117E 02	*	*	*	*	*	*
0.146E 02	*	*	*	*	*	*
0.175E 02	*	*	*	*	*	*
0.205E 02	*	*	*	*	*	*
0.234E 02	*	*	*	*	*	*
0.263E 02	*	*	*	*	*	*
0.292E 02	*	*	*	*	*	*
0.322E 02	*	*	*	*	*	*
0.351E 02	*	*	*	*	*	*
0.380E 02	*	*	*	*	*	*
0.409E 02	*	*	*	*	*	*
0.438E 02	*	*	*	*	*	*
0.468E 02	*	*	*	*	*	*
0.497E 02	*	*	*	*	*	*
0.526E 02	*	*	*	*	*	*
0.555E 02	*	*	*	*	*	*
0.585E 02	*	*	*	*	*	*
0.614E 02	*	*	*	*	*	*
0.643E 02	*	*	*	*	*	*
0.672E 02	*	*	*	*	*	*
0.702E 02	*	*	*	*	*	*
0.731E 02	*	*	*	*	*	*
0.760E 02	*	*	*	*	*	*
0.789E 02	*	*	*	*	*	*
0.819E 02	*	*	*	*	*	*
0.848E 02	*	*	*	*	*	*
0.877E 02	*	*	*	*	*	*
0.895E 02	*	*	*	*	*	*
0.913E 02	*	*	*	*	*	*
0.931E 02	*	*	*	*	*	*
0.950E 02	*	*	*	*	*	*
0.968E 02	*	*	*	*	*	*
0.586E 02	*	*	*	*	*	*
0.10CE 03	*	*	*	*	*	*
0.132E 03	*	*	*	*	*	*
0.104E 03	*	*	*	*	*	*
0.106E 03	*	*	*	*	*	*
0.108E 03	*	*	*	*	*	*
0.109E 03	*	*	*	*	*	*
0.111E 03	*	*	*	*	*	*
0.113E 03	*	*	*	*	*	*
0.115E 03	*	*	*	*	*	*
0.117E 03	*	*	*	*	*	*
0.119E 03	*	*	*	*	*	*
0.120E 03	*	*	*	*	*	*
0.122E 03	*	*	*	*	*	*
0.124E 03	*	*	*	*	*	*
0.125E 03	*	*	*	*	*	*
0.127E 03	*	*	*	*	*	*
0.128E 03	*	*	*	*	*	*
0.130E 03	*	*	*	*	*	*
0.131E 03	*	*	*	*	*	*
0.132E 03	*	*	*	*	*	*
0.134E 03	*	*	*	*	*	*
0.135E 03	*	*	*	*	*	*
0.137E 03	*	*	*	*	*	*
0.138E 03	*	*	*	*	*	*
0.139E 03	*	*	*	*	*	*
0.141E 03	*	*	*	*	*	*
0.142E 03	*	*	*	*	*	*
0.144E 03	*	*	*	*	*	*
0.145E 03	*	*	*	*	*	*
0.146E 03	*	*	*	*	*	*
0.148E 03	*	*	*	*	*	*
0.149E 03	*	*	*	*	*	*
0.151E 03	*	*	*	*	*	*
0.152E 03	*	*	*	*	*	*
0.154E 03	*	*	*	*	*	*
0.155E 03	*	*	*	*	*	*
0.157E 03	*	*	*	*	*	*
0.159E 03	*	*	*	*	*	*

PLOT OF GROUP 2 / GROUP 3

Z-AXIS

X-AXIS	0.104E 01	0.138E 01	0.172E 01	0.205E 01	0.239E 01	0.273E 01
0.0
0.292E 01
0.585E 01
0.877E 01
0.117E 02
0.146E 02
0.175E 02
0.205E 02
0.234E 02
0.263E 02
0.292E 02
0.322E 02
0.351E 02
0.380E 02
0.409E 02
0.438E 02
0.468E 02
0.497E 02
0.526E 02
0.555E 02
0.585E 02
0.614E 02
0.643E 02
0.672E 02
0.702E 02
0.731E 02
0.760E 02
0.789E 02
0.819E 02
0.848E 02
0.877E 02
0.895E 02
0.913E 02
0.931E 02
0.950E 02
0.968E 02
0.986E 02
C.100E 03
0.102E 03
0.104E 03
0.106E 03
0.108E 03
0.109E 03
0.111E 03
0.113E 03
0.115E 03
0.117E 03
0.119E 03
0.120E 03
0.122E 03
0.124E 03
0.125E 03
0.127E 03
0.128E 03
0.130E 03
0.131E 03
0.132E 03
0.134E 03
0.135E 03
0.137E 03
0.138E 03
0.139E 03
0.141E 03
0.142E 03
0.144E 03
0.145E 03
0.146E 03
0.148E 03
0.149E 03
0.151E 03	*	*	*	*	*	*
0.152E 03	*	*	*	*	*	*
0.154E 03	*	*	*	*	*	*
0.155E 03	*	*	*	*	*	*
0.157E 03	*	*	*	*	*	*
0.159E 03	*	*	*	*	*	*

GROUP 1 / GROUP 2

0.12327E 01	0.12327E 01	0.12327E 01	0.12327E 01	C.12327E 01
0.12327E 01	0.12327E 01	0.12328E 01	0.12328E 01	0.12328E 01
0.12328E 01				
0.12329E 01	0.12329E 01	0.12329E 01	0.12329E 01	C.12329E 01
0.12329E 01	0.12330E 01	C.12330E 01	0.12330E 01	0.12330E 01
0.12330E 01	0.12331E 01	0.12331E 01	0.12331E 01	0.12331E 01
0.12331E 01	0.12331E 01	0.12332E 01	0.12332E 01	0.12332E 01
0.12332E 01				
0.12333E 01				
C.12333F 01	0.12333E 01	0.12333E 01	0.12333E 01	0.12333E 01
0.12333E 01	0.12334E 01	0.12334E 01	C.12334E 01	0.12334E 01
0.12334E 01	0.12334E 01	C.12334E 01	0.12334E 01	0.12335E 01
0.12335E 01	0.12335E 01	0.12337E 01	0.12339E 01	0.12343E 01
0.12347E 01	0.12351E 01	0.12349E 01	0.12324E 01	0.12217E 01
0.11858E 01	0.1446E 01	0.11485E 01	C.12348E 01	0.15979E 01

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	0.114E 01	0.124E 01	Z-AXIS 0.133E 01	0.142E 01	0.151E 01	0.160E 01
R-AXIS	*	*	*	*	*	*
0.0	*	*	*	*	*	*
0.292E 01	*	*	*	*	*	*
0.585E 01	*	*	*	*	*	*
0.877E 01	*	*	*	*	*	*
0.117E 02	*	*	*	*	*	*
0.146E 02	*	*	*	*	*	*
0.175E 02	*	*	*	*	*	*
0.205E 02	*	*	*	*	*	*
0.234E 02	*	*	*	*	*	*
0.263E 02	*	*	*	*	*	*
0.292E 02	*	*	*	*	*	*
0.322E 02	*	*	*	*	*	*
0.351E 02	*	*	*	*	*	*
0.380E 02	*	*	*	*	*	*
0.409E 02	*	*	*	*	*	*
0.438E 02	*	*	*	*	*	*
0.468E 02	*	*	*	*	*	*
0.497E 02	*	*	*	*	*	*
0.526E 02	*	*	*	*	*	*
0.555E 02	*	*	*	*	*	*
0.585E 02	*	*	*	*	*	*
0.614E 02	*	*	*	*	*	*
0.643E 02	*	*	*	*	*	*
0.672E 02	*	*	*	*	*	*
0.702E 02	*	*	*	*	*	*
0.731E 02	*	*	*	*	*	*
0.760E 02	*	*	*	*	*	*
0.789E 02	*	*	*	*	*	*
0.814E 02	*	*	*	*	*	*
0.848E 02	*	*	*	*	*	*
0.877E 02	*	*	*	*	*	*
0.895E 02	*	*	*	*	*	*
0.913E 02	*	*	*	*	*	*
0.931E 02	*	*	*	*	*	*
0.950E 02	*	*	*	*	*	*
0.968E 02	*	*	*	*	*	*
0.986E 02	*	*	*	*	*	*
0.100E 03	*	*	*	*	*	*
0.102E 03	*	*	*	*	*	*
0.104E 03	*	*	*	*	*	*
0.106E 03	*	*	*	*	*	*
0.108E 03	*	*	*	*	*	*
0.109E 03	*	*	*	*	*	*
0.111E 03	*	*	*	*	*	*
0.113E 03	*	*	*	*	*	*
0.115E 03	*	*	*	*	*	*
0.117E 03	*	*	*	*	*	*
0.119E 03	*	*	*	*	*	*
0.120E 03	*	*	*	*	*	*
0.122E 03	*	*	*	*	*	*
0.124E 03	*	*	*	*	*	*
0.125E 03	*	*	*	*	*	*
0.127E 03	*	*	*	*	*	*
0.128E 03	*	*	*	*	*	*
0.130E 03	*	*	*	*	*	*
0.131E 03	*	*	*	*	*	*
0.132E 03	*	*	*	*	*	*
0.134E 03	*	*	*	*	*	*
0.135E 03	*	*	*	*	*	*
0.137E 03	*	*	*	*	*	*
0.138E 03	*	*	*	*	*	*
0.139E 03	*	*	*	*	*	*
0.141E 03	*	*	*	*	*	*
0.142E 03	*	*	*	*	*	*
0.144E 03	*	*	*	*	*	*
0.145E 03	*	*	*	*	*	*
0.146E 03	*	*	*	*	*	*
0.148E 03	*	*	*	*	*	*
0.149E 03	*	*	*	*	*	*
0.151E 03	*	*	*	*	*	*
0.152E 03	*	*	*	*	*	*
0.154E 03	*	*	*	*	*	*
0.155E 03	*	*	*	*	*	*
0.157E 03	*	*	*	*	*	*
0.159E 03	*	*	*	*	*	*

POWER OUTPUT IN REGIONS 1,2,3

0.13406E 10 0.77863E 09 0.32180E 09

POWER DENSITY IN REGIONS 1,2,3 (KW/LITER)

0.15171E 03 0.13348E 01 0.54903E 00